



## Calculation of Plate Temperatures in A MK 4 LEU Fuel Element

Haack, K.

*Publication date:*  
1988

*Document Version*  
Publisher's PDF, also known as Version of record

[Link back to DTU Orbit](#)

*Citation (APA):*  
Haack, K. (1988). *Calculation of Plate Temperatures in A MK 4 LEU Fuel Element*. Risø National Laboratory. Risø-M No. 2745

---

### General rights

Copyright and moral rights for the publications made accessible in the public portal are retained by the authors and/or other copyright owners and it is a condition of accessing publications that users recognise and abide by the legal requirements associated with these rights.

- Users may download and print one copy of any publication from the public portal for the purpose of private study or research.
- You may not further distribute the material or use it for any profit-making activity or commercial gain
- You may freely distribute the URL identifying the publication in the public portal

If you believe that this document breaches copyright please contact us providing details, and we will remove access to the work immediately and investigate your claim.

# Calculation of Plate Temperatures in A Mk 4 LEU Fuel Element

Karsten Haack

Risø National Laboratory, DK-4000 Roskilde, Denmark  
September 1988

RISØ-M-2745

CALCULATION OF PLATE TEMPERATURES IN A MK 4 LEU FUEL ELEMENT.

Karsten Haack

Abstract. A calculation method for estimating the axial temperature distributions of each of the 26 fuel elements of the DR 3 core is described and demonstrated. With input data for fuel element power, D<sub>2</sub>O outlet temperature and main D<sub>2</sub>O circulator combination, a computer code calculates all important temperatures in the fuel element.

September 1988

Risø National Laboratory, DK-4000 Roskilde, Denmark

**ISBN 87-550-1462-3**

**ISSN 0418-6435**

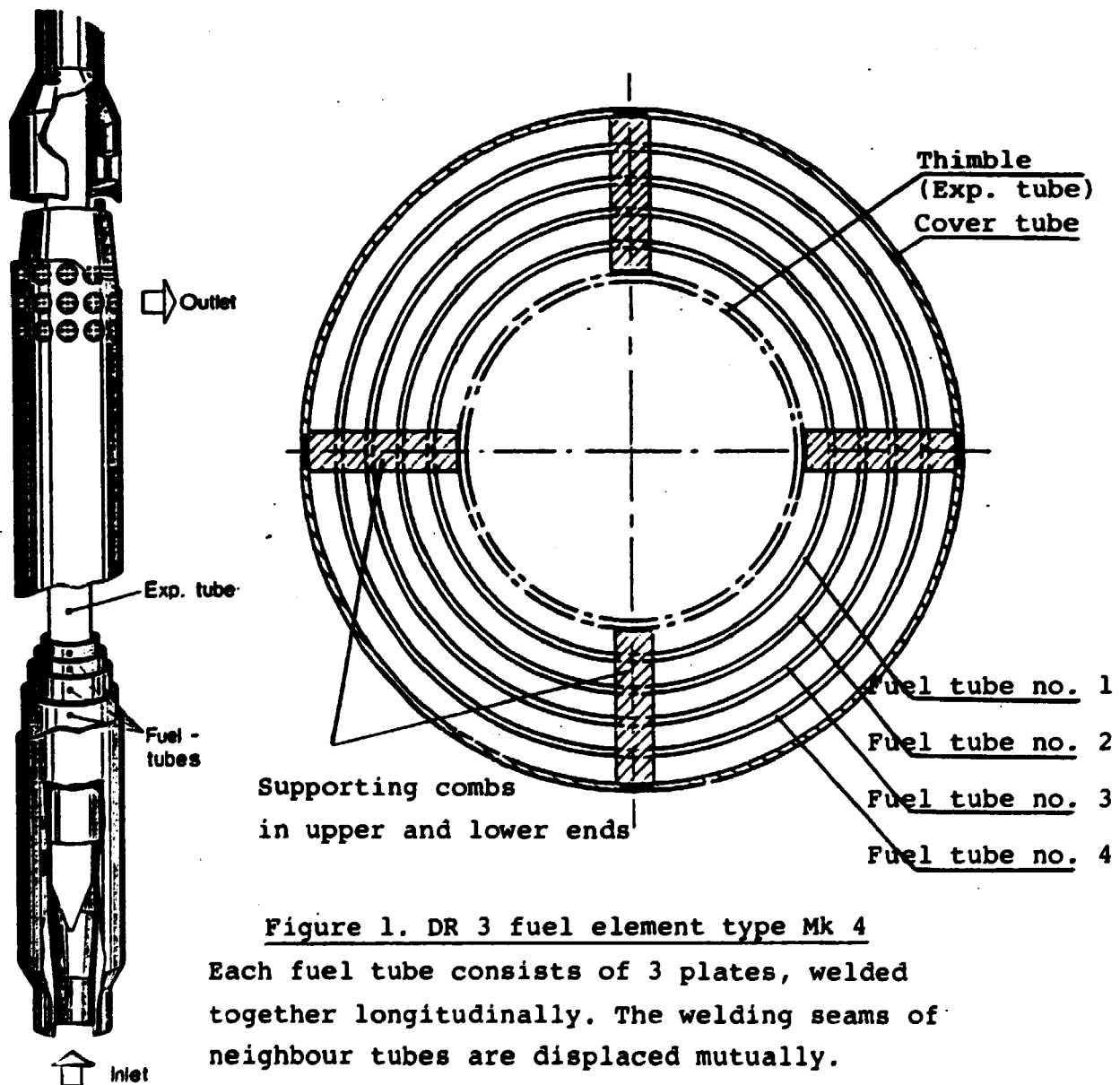
**Grafisk Service, Risø 1988**

LIST OF CONTENTS	PAGE
Introduction .....	5
1. Power release and heat deposition in the fuel tubes .....	7
2. Flow- and power-distributions between the fuel tubes .....	8
3. Fuel meat-to-coolant heat transfer calculations.	8
4. The D <sub>2</sub> O temperature rise by passing through the cooling channels .....	10
5. The total thermo-hydraulic pattern in a fuel element .....	11
6. Acknowledgements .....	15
7. References .....	15
Figures .....	16
Appendix 1 .....	19
Appendix 2 .....	24

## Introduction

In order to provide a means of compensation for avoiding reduction of the thermal neutron flux densities in the core positions of DR 3 while converting from highly enriched to low enriched uranium, application to the danish authorities has been made for upgrading the reactor power level.

This application involves a recalculation of the temperature distributions of the fuel elements. Earlier calculations have been done on the box-type Mk 2 fuel element. By the results from new flow measurement in the annular-tube-type Mk 4 fuel elements in 1986 a prerequisite was provided for exact temperature calculations.



# 1. Power release and heat deposition in the fuel tubes

The fission power release in the fuel tubes can be calculated by means of the DR 3-SIM code <sup>1)</sup>. The results are given in kW per cm axial length for all four fuel tubes together.

However, a part of this heat release is not absorbed in the fuel tubes. Recent references <sup>8)</sup> divide the release of fission energy from <sup>235</sup>U as follows:

Kinetic energy of fission fragments	166,2 MeV
Instantaneous gamma-ray energy	8,0 -
Kinetic energy of fission neutrons	4,8 -
Beta particles from fission products	7,0 -
Gamma-rays from fission products	7,2 -
Neutron reactions in core and coolant	2,3 - <sup>*)</sup>
<hr/>	
Total fission energy	195,5 MeV

<sup>\*)</sup> calculated for the DR 3 core.

The energy from fission fragments and from beta particles will, almost totally, be absorbed in the fuel tubes, but only a fraction  $\sim 0,2$  of the gamma-ray energy and fission neutron energy will be absorbed here. Thus the deposition of energy in the fuel tubes will be:

Kinetic energy of fission fragments	166,2 MeV
Instantaneous gamma-ray energy	0,5 -
Kinetic energy of fission neutrons	0,5 -
Beta particles from fission products	6,1 -
Gamma-rays from fission products	0,5 -
Neutron reactions in core and coolant	2,1 -
<hr/>	
	175,9 MeV

The total energy absorption in the fuel tubes is consequently estimated at 175,9 MeV per fission, which means that a fraction of  $\mathcal{K} = \frac{175,9}{195,5} = 0,90$  of the total energy released is actually absorbed in the fuel tubes themselves.

## 2. Flow- and power-distributions between the fuel tubes

The flow distribution between the fuel elements in the core was measured in 1986 <sup>4)</sup> in each of the 3 combinations of the main circulators. (2 of the 3 main circulators are operating at a time). See table 6 in appendix 1.

The flow distribution between the 5 cooling channels in a Mk 4 fuel element has been measured in Jülich, GDR. The results have been used by Kaiser <sup>5)</sup> for calculation of the flow stability conditions in Mk 4 fuel elements. See table 4 in app. 1.

The power generation distribution between the fuel tubes has been calculated by means of the code DR 3/SIM for Mk 4 fuel elements with 4 fuel tubes as well as 3 fuel tubes, see table 5 in app. 1.

## 3. Fuel meat-to-coolant heat transfer calculations

The radial heat transfer path is divided in three steps:

- I) - Through the fuel meat
- II) - Through the cladding,
- III) - Through the D<sub>2</sub>O boundary layer at the cladding surface.

I). The heat conduction through the heat-generating meat of the fuel tube is calculated by means of the equation <sup>2)</sup>:

$$T_0 - T_x = \frac{q \cdot \rho_k}{2 \lambda_k} x^2 \quad (1)$$

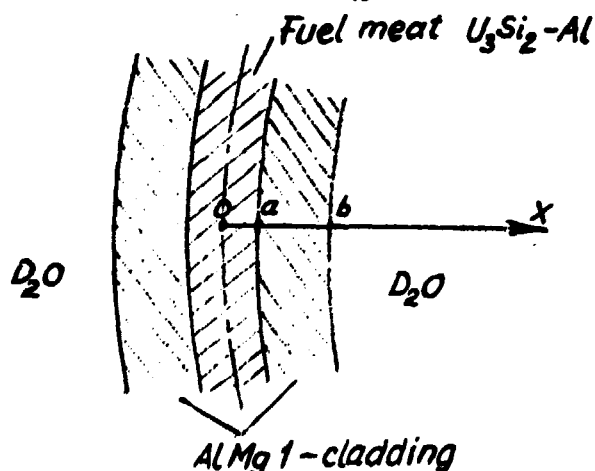


Fig. 2. Fuel tube cross section

where T is the temperature (°C)

q is the heat deposition (Wkg<sup>-1</sup>)

$\rho_k$  is the meat density (kgm<sup>-3</sup>)

$\lambda_k$  is the thermal conductivity (Wm<sup>-1</sup> °C<sup>-1</sup>)

The fuel tubes are considered as plates in these calculations because the tube wall thickness is small compared to the tube diameters.



The power release  $P_z$  as calculated by means of the DR 3/SIM code is given in units of ( $\text{kW}\cdot\text{cm}^{-1}$ ) in 8 intervals of the axial fuel element length.

The relation between  $q$  and  $P_z$  ( $\text{W}\cdot\text{m}^{-1}$ ) is

$$q = \frac{\lambda_n \cdot \Delta_n \cdot P_z}{2 \cdot a \cdot b_{k,n} \cdot \rho_k} = \frac{\lambda_n \cdot \Delta_n \cdot l_k}{A_{k,n} \cdot a \cdot \rho_k} \cdot P_z \quad (2)$$

where  $\Delta_n$  is the fraction of heat deposition in tube no.  $n$

$b_{k,n}$  (m) is the total of the 3 plate widths of tube no.  $n$

$l_k$  (m) is the axial fuel meat length

$A_{k,n} = 2 \cdot b_{k,n} \cdot l_k$  is the effective heat transfer surface of tube no.  $n$  ( $\text{m}^2$ )

By substitution of  $q$  from eq. (2) in eq. (1) we obtain at  $x=a$ :

$$T_o - T_a = \frac{\Delta_n \cdot l_k \cdot a \cdot \lambda}{A_{k,n} \cdot 2 \cdot \lambda_k} P_z \quad (3)$$

II) The temperature drop through the cladding is:

$$T_a - T_b = \frac{\dot{Q}(b-a)}{\lambda_{Al}} = \frac{\Delta_n (b-a) \cdot l_k}{A_{k,n} \cdot \lambda_{Al}} \lambda P_z \quad (4)$$

where  $\lambda_{Al}$  is the thermal conductivity of aluminium ( $\text{Wm}^{-1} \text{ } ^\circ\text{C}^{-1}$ )

$\dot{Q}$  is the heat flow ( $\text{Wm}^{-2}$ )

and  $a$  (see fig. 2)

III) The temperature drop through the  $\text{D}_2\text{O}$  film layer close to the cladding surface can be calculated according to 6):

$$T_b - T_{\text{D}_2\text{O}} = \frac{\dot{Q} \cdot d_h}{\lambda_D \cdot \text{Nu}_b} = \frac{\Delta_n \cdot d_h \cdot l_k \cdot \lambda}{A_{k,n} \cdot \lambda_D \cdot \text{Nu}_b} \cdot P_z \quad (5)$$

where  $d_h$  is the hydraulic diameter of the cooling channel (m)

$$d_h = \frac{4 \cdot \text{cross section of cooling channel}}{\text{wetted perimeter}}$$

$\lambda_D$  is the thermal conductivity of  $\text{D}_2\text{O}$  ( $\text{Wm}^{-1} \text{ } ^\circ\text{C}^{-1}$ )

and  $\text{Nu}_b$  is the Nusselt-number for annular gaps, given by the

equation:

$$Nu_b = \frac{0.86 \left(\frac{d_i}{d_y}\right)^{0.84} + (1 - 0.14 \cdot \left(\frac{d_i}{d_y}\right)^{0.6})}{1 + \frac{d_i}{d_y}} \cdot \frac{\xi \cdot (Re - 1000) \cdot Pr \cdot \left(1 + \left(\frac{d_i}{d_k}\right)^{\frac{2}{3}}\right)}{1 + 12.7 \sqrt{\frac{\xi}{8} \cdot (Pr^{\frac{2}{3}} - 1)}} \left(\frac{Pr}{Pr_w}\right)^{0.11} \quad (6)$$

where  $d_i$  and  $d_y$  are the inner and outer diameter of the cooling channel,

$Re$  is the Reynolds number  $Re = \frac{v_z \cdot d_h}{\nu}$  (7)

$v_z$  is the coolant mean velocity ( $ms^{-1}$ )

$\nu$  is the kinematic viscosity of the  $D_2O$  film layer ( $m^2s^{-1}$ )

$\xi$  is the pressure drop coefficient

$$\xi = (1.82 \cdot \log_{10} (Re) - 1.64)^{-2} \quad (8)$$

$Pr$  is the Prandtl number for the  $D_2O$  coolant

$Pr_w$  is the Prandtl number for the film layer close to the wall.

#### 4. The $D_2O$ temperature rise by passing through the cooling channels

By means of the methods outlined in section 3 every fuel meat temperature can be calculated assumed the adjacent  $D_2O$  coolant temperature is known. The temperature distribution of the  $D_2O$  coolant in the cooling channels of a fuel element is given by the equation:

$$\overline{T_{D_2O,z}} - T_i = \int_{-0.3}^z \frac{P_z dz}{4.187 \cdot v_m} = \frac{1}{4.187 \cdot F_m} \int_{-0.3}^z P_z dz \quad (9)$$

where  $P_z$  ( $Wm^{-1}$ ) is the axial power deposition in the interval  $dz$  and  $F_m$  ( $kg s^{-1}$ ) is the mean coolant mass flow through the element.

This is a mean of all 5 cooling channels. In the cooling channel between tube no.  $n$  and tube no.  $(n-1)$  the temperature distribution will be:

$$T_{D_2O,z} - T_i = \frac{A_n + A_{n-1}}{2 \cdot 4.187 \cdot \delta_n \cdot F_m} \int_{-0.3}^z P_z dz \quad (10)$$

where  $\delta_n$  is the flow ratio in the cooling channel inside tube no. n. ( $F_z = \delta_n \cdot F_m$ )

and  $A_n$  is the power deposition ratio of tube no. n.

The values of  $\delta_n$  and  $A_n$  are given in appendix 1, tables 4 and 5.

#### 5. The total thermo-hydraulic pattern in a fuel element

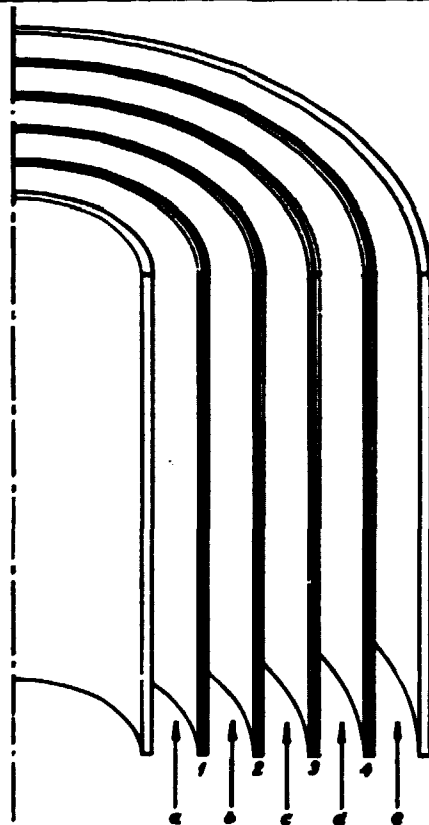


Fig. 3. Fuel tubes and cooling channels in a Mk 4 fuel element with 4 fuel tubes

It is obvious that the temperature rises through the inner and outer channels (a and e) are lowest as these are heated from one side, only. Tube no. 1 and 4 will, consequently, be cooled better and obtain lower temperatures than tube no. 2 and 3. This will cause lower temperatures in the cooling channels b and d, compared with c.

Thus, the heat transport to the inside of each tube is different from that to the outside, so the presumption in chapter 3: That the temperature maximum is situated in the center of the fuel meat (see fig. 2) does not hold.

However, as the temperature drop through the fuel meat itself is very small, a reasonable good approximation is obtained by sticking to the presumption of max. temperature in the meat centre, but assuming different heat transports to the two sides.

Denoting the ratios of heat removal from tube no. n outwards  $\beta_{n,o}$  and inwards  $\beta_{n,i}$ , the heat transport to the cooling channel between tube no. n and tube no. n+1 will be:

$$P_z \cdot \Delta_n \cdot \beta_{n,o} + P_z \cdot \Delta_{(n+1)} \cdot \beta_{n+1,i} \quad (11)$$

which will modify eqs. (10), (3), (4) and (5):

$$T_{D_2O,z} - T_i = \frac{\Delta_n \cdot \beta_{n,o} + \Delta_{n+1} \cdot \beta_{n+1,i}}{4.187 \cdot \delta_n \cdot F_m} \int_{-0.3}^z P_z dz \quad (12)$$

$$T_o - T_a = \frac{\Delta_n \cdot l_k \cdot a}{A_{k,n} \cdot \lambda_k} \cdot 2 \cdot \beta \cdot P_z \quad (13)$$

$$T_a - T_b = \frac{\Delta_n \cdot (b-a) \cdot l_k}{A_{k,n} \cdot \lambda_{Al}} \cdot 2 \cdot \beta \cdot P_z \quad (14)$$

$$T_b - T_{D_2O} = \frac{\Delta_n \cdot d_h \cdot l_k}{A_{k,n} \cdot \lambda_D \cdot Nu_b} \cdot 2 \cdot \beta \cdot P_z \quad (15)$$

As  $\beta_{n,i} + \beta_{n,o} = 1$ , the mean value of the  $\beta$ 's is 0.5. Consequently the factor 2 had to be introduced in eqs. (12), (3), (4) and (5) in order to obtain the correct result from eqs. (12), (13), (14) and (15).

The calculation of the whole temperature distribution in

a fuel element is carried out under the assumptions:

- a) The  $\beta$ -values are independent of  $z$ , i.e. they doesn't vary along the length of the fuel element (this is tested later).
- b) The thermal-hydraulic conditions are symmetric around the vertical axis of the fuel element.

The temperatures can be calculated by iterations:

By choosing a suitable set of the 8  $\beta$ -values, observing that  $\beta_{n,i} + \beta_{n,o} = 1$ , the temperatures at the core-centre plane (CCP) where  $z = 0$  can be estimated by means of equations (12), (13), (14) and (15).

The meat temperature calculated from inside a fuel tube should equal that calculated from outside the tube. If this is not the case, a new set of  $\beta$ -values must be chosen, and the calculation procedure must be repeated until the same meat centre temperature is obtained in each tube by calculations from both sides of the tube.

The iterations have been carried out on a computer using the data given in appendix 1. The final set of  $\beta$ -values at CCP were:

Tube no. n	1	2	3	4
$\beta_{n,i}$	0.638	0.523	0.476	0.356
$\beta_{n,o}$	0.362	0.477	0.524	0.644

In order to show the significance of the error introduced by the assumption a), the temperature distributions in fuel element C2 in reactor cycle 325 has been calculated from inside as well as from outside of each fuel tube. The results are shown on figure 4. The two curves for each tube coincides

at  $z = 0$ , which was aimed by the iteration calculation. In the upper and lower ends of the fuel tubes a difference between the outer and inner calculations is noticed, in particular for tubes no. 1 and 4, but the maximum difference is below  $2^{\circ}\text{C}$ . By representing the true meat temperature by the mean from the inside and outside calculations, the error is well below  $1^{\circ}\text{C}$ , which means that the legitimacy of assumption a) seems to be justified.

The last question is: Does the set of  $\beta$ -values in the table above apply to all positions in the core? It is calculated for a LEU 180g fuel element in the core position C2 with a flow 13.20 kg/s and 1000 kW fuel element power. Using the same set of  $\beta$ -values, calculations of meat temperatures from outside and inside of all 4 fuel tubes have been carried out for the same reactor cycle 327 with LEU 180g fuel elements in core position B2 (coolant flow: 13.60 kg/s) and in A1 (flow: 16.20 kg/s), both for 1000 kW element power. The results are shown on figs. 5 and 6. It is seen that the curves crosses close to  $z = 0$  and that the max. difference in the ends is less than  $2^{\circ}\text{C}$ .

All 26 fuel elements flow are between 12.00 and 16.20 kg/s in the 3 combinations of 2 running main circulators 1P1/1,2,3. Only 9 of these 3 x 26 flow values are below that of C2: 13.20 kg/s. As furthermore the  $\beta$ -factors are nearly independent of the element power, it seems to be reasonably justified to use the same set of  $\beta$ -values for all core positions.

As an example, the fuel tube temperature distributions in all 26 core positions have been calculated by means of the method outlined above. The calculations refer to the known conditions of reactor cycle no. 340. The graphs are shown in appendix 2. The maximum meat temperature is  $94.8^{\circ}\text{C}$  in tube no. 3 of the core position C2, which contained a new 150g HEU element generating 543 kW. The reactor power was 10.0 MW. The presumed  $\text{D}_2\text{O}$  bulk temperature in the reactor tank was  $70^{\circ}\text{C}$ , which is the maximum allowable bulk temperature.

## 6. Acknowledgements

I wish to thank mr. P. Wiig for doing the computer calculations and preparing the figures, and miss Anni Lambæk for typing the report. The kind assistance of proff. J. Bukovsky, Msc., Plzen University, CSSR, for revising the report is highly appreciated.

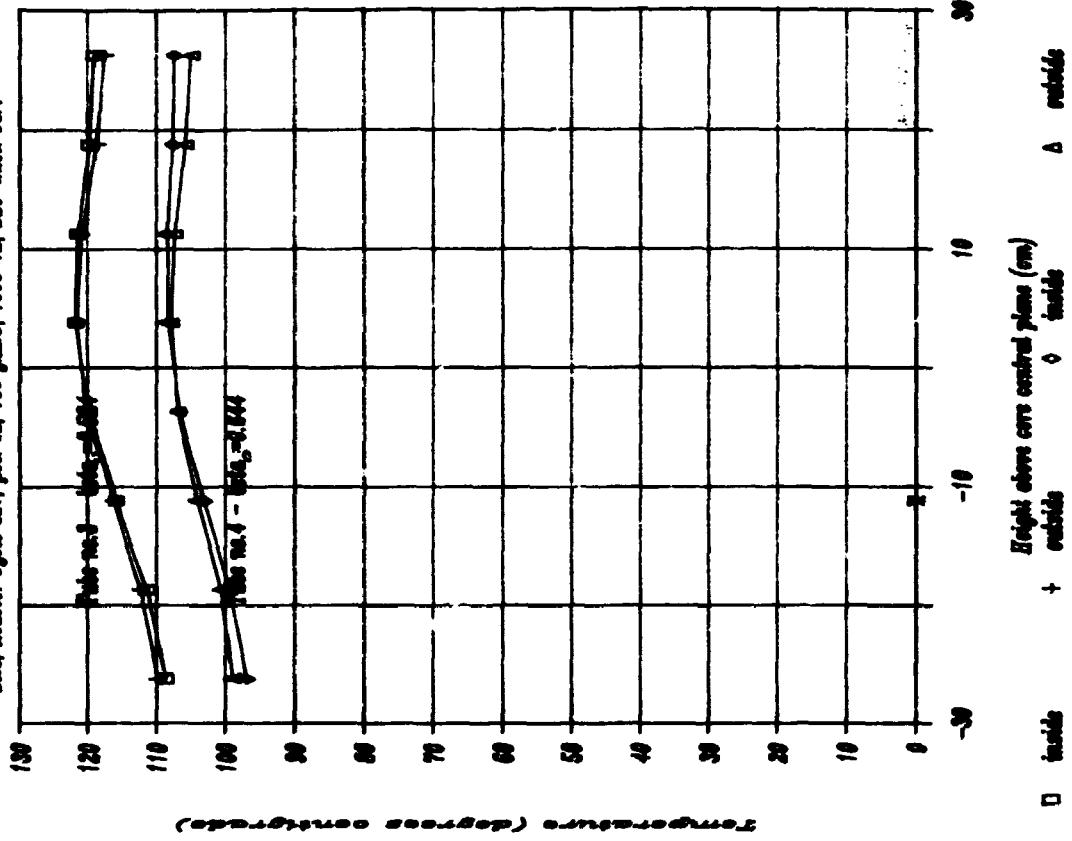
## 7. References

1. Erik Nonbøl, Development of a Model for the Danish Research Reactor DR 3, RP-05-85, Risø National Laboratory, March 1985.
2. Samuel Glasstone, Principles of Nuclear Reactor Engineering, Macmillan & Co, Ltd., London 1956.
3. Jens Qvist, Flowinstabilitetsfaktorer for LEU kerne baseret på flowmålinger fra juni 1986, DR 3-17/M 2006, Risø, September 1987
4. P.R.Winstrøm, Flowmeasurements in the DR 3 core. DR 3-01/M 1967, Risø, March 1987.
5. N.E. Kaiser, Teoretisk undersøgelse af varme- og strømningstekniske forhold i DR 3 med Mk 4 brændselseleranter, Risø-M-916, Risø, Juni 1969.
6. VDI Wärmeatlas, Verein Deutscher Ingenieure, VDI-Verlag, GmbH, Düsseldorf 1977.
7. CRC Handbook of Chemistry and Physics, CRC Press, Inc. Cleveland, Ohio, USA, 1977.
8. James, M.F., "Energy Released in Fission", AEEW-M863, 1969.

FIG. 4

# Meat Temperature.

DS3, Reaction Cycle 377, pos C2, 100 g LBS, 1000 km, DS3-inlet: 63.4 °



# Meat Temperature.

DS3, Reaction Cycle 377, pos C2, 100 g LBS, 1000 km, DS3-inlet: 63.4 °

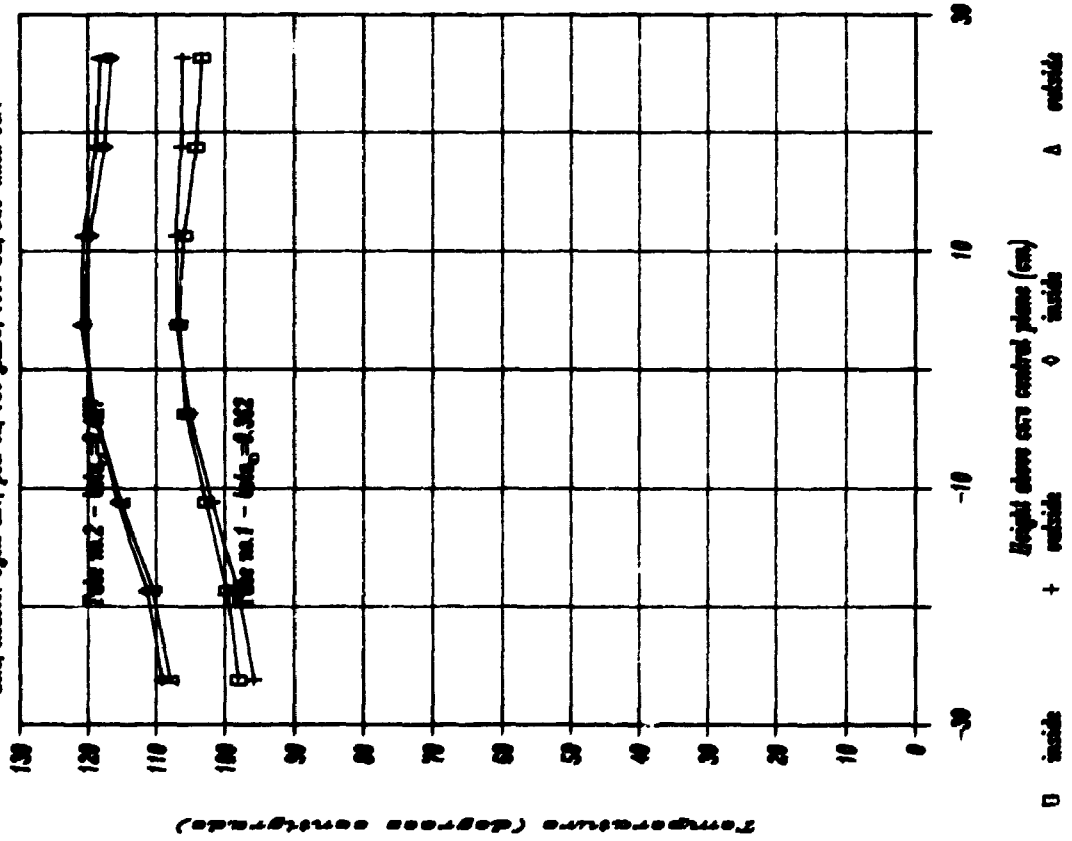
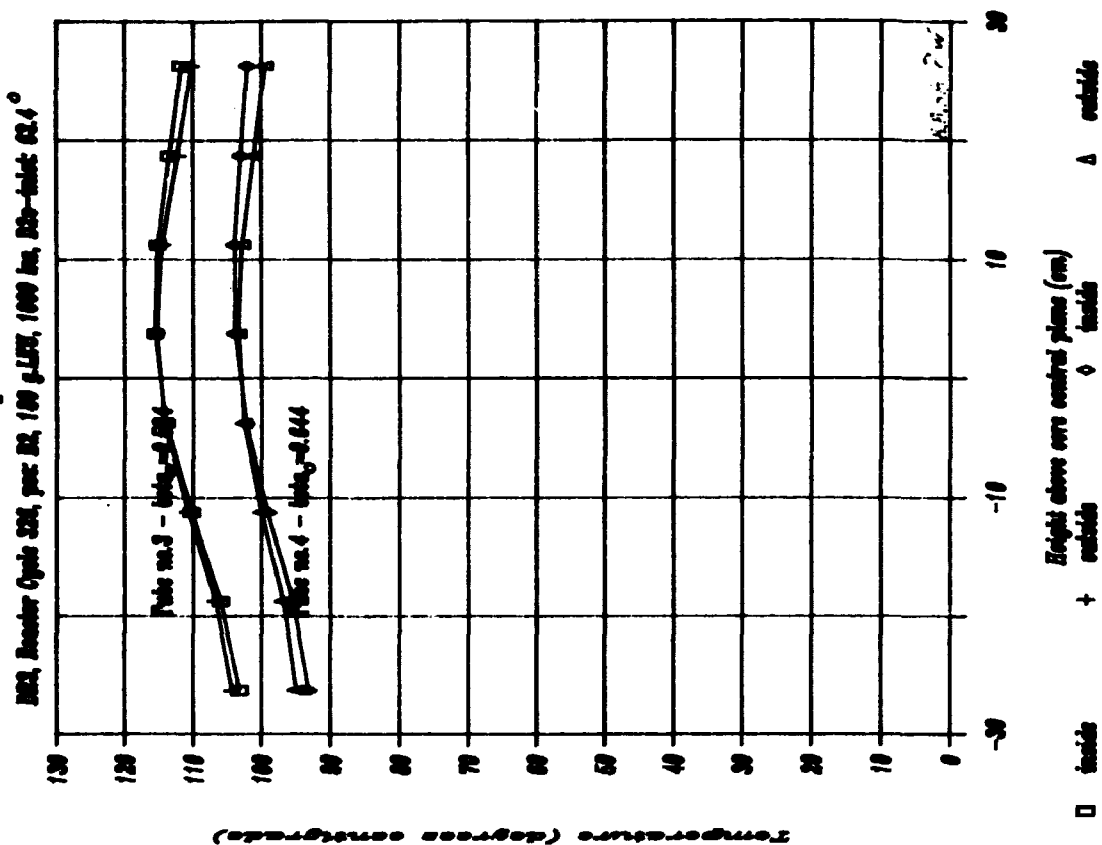




FIG. 5

# Meat Temperature.



# Meat Temperature.

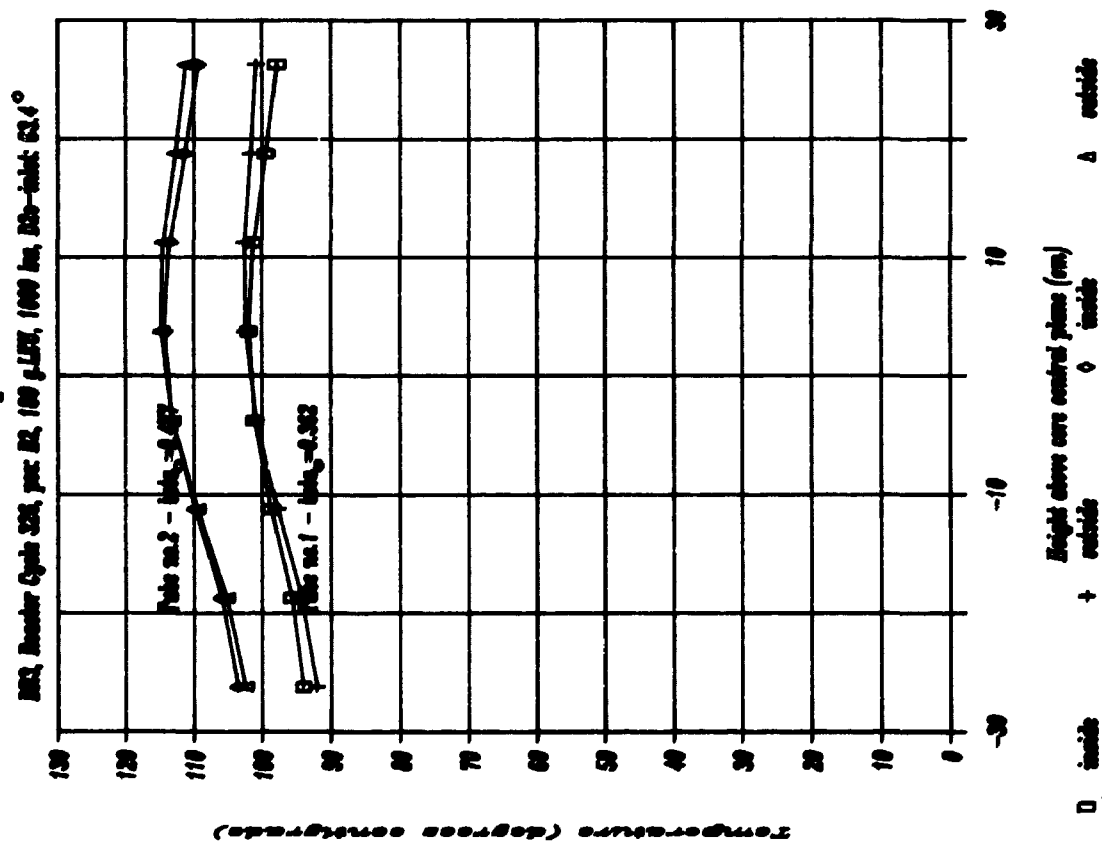
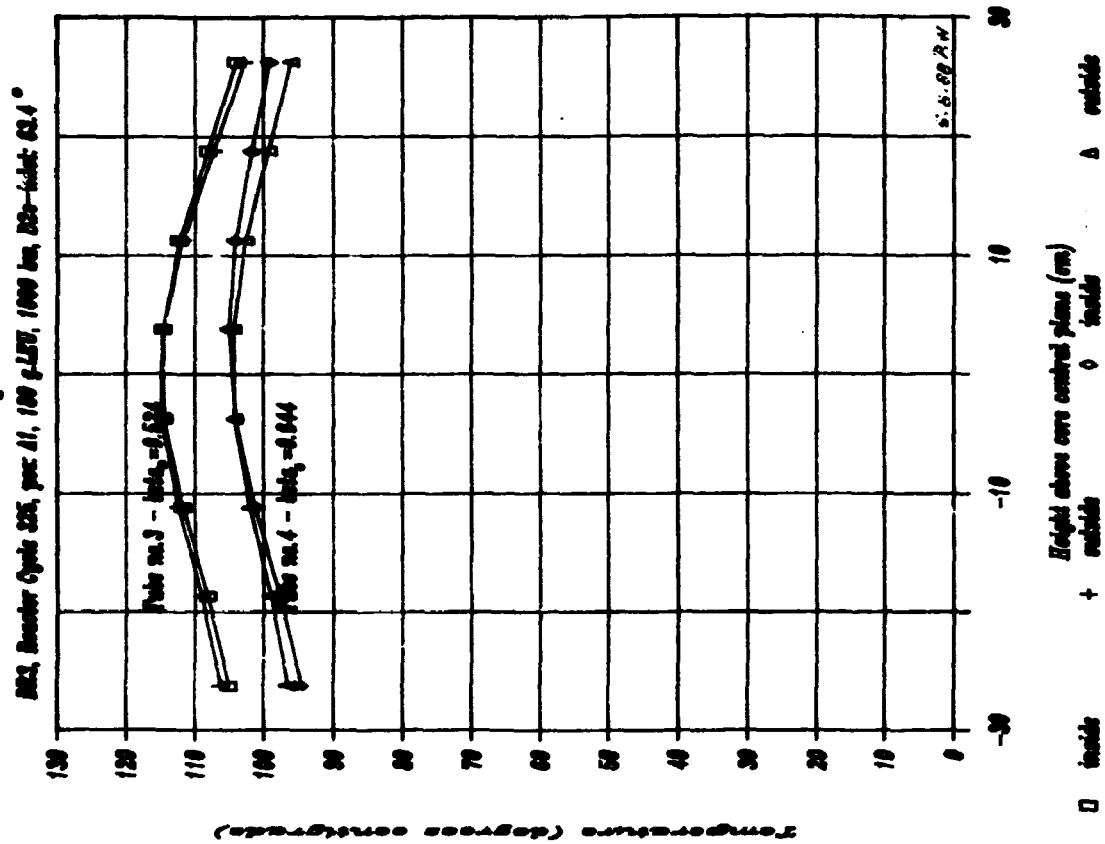
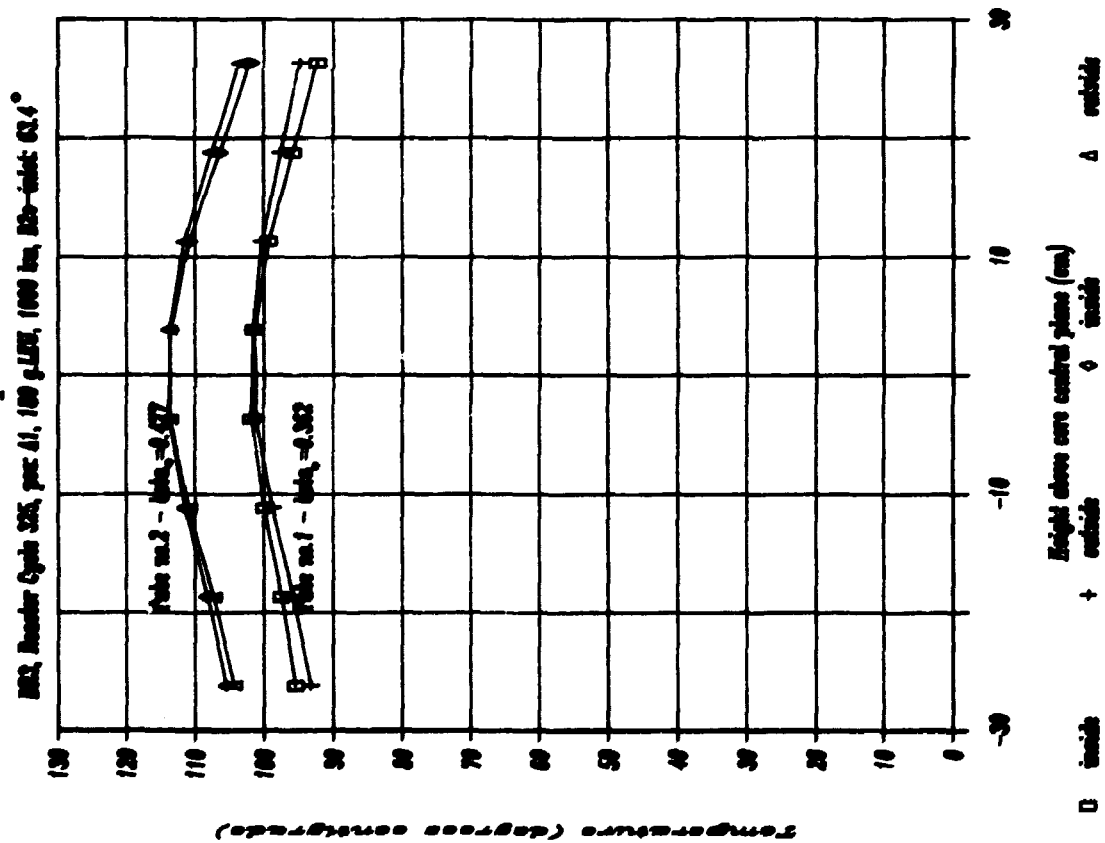


FIG. 6

# Meat Temperature.



# Meat Temperature.



Al/1

APPENDIX 1

Heat transmission data

$T_{D_2O}$ (°C)	$\nu$ ( $10^{-6} m^2 s^{-1}$ )	$\rho_D^{*)}$ ( $Mg m^{-3}$ )	$\lambda_D$ ( $10^{-3} W m^{-1} °C^{-1}$ )	Pr	Pr <sub>w</sub>
50	0.551	1.09576	643	3.54	
60	0.471	1.09060	654	2.96	
70	0.409	1.08475	662	2.53	
80	0.361	1.07824	670	2.20	
90	0.322	1.07112	676	1.94	
100	0.291	1.06346	681	1.73	0.984
110				1.56	1.000
120				1.42	1.02
130				1.31	1.04
140				1.21	1.06
150				1.14	1.08
160				1.07	1.11

Table 1. Heavy water data acc. to 6)

\*) data from 7)

Channel between tube no.	a (0-1)	b (1-2)	c (2-3)	d (3-4)	e (4-5)
$d_i$ ( $10^{-3} m$ )	54.05	63.85	73.65	83.45	93.25
$d_y$ ( $10^{-3} m$ )	60.85	70.65	80.45	90.25	100.05
$\frac{d_i}{d_y}$	0.8883	0.9038	0.9155	0.9247	0.9320
$d_h$ ( $10^{-3} m$ )	14.41	7.73	7.60	7.50	15.60
$v_z$ ( $ms^{-1}$ )	3.10	2.95	2.71	2.91	2.79
$l_k$ (m)	0.600	0.600	0.600	0.600	0.600

Table 2. Cooling channel data

Al/2

	Meat	Cladding
$\rho$ (Mgm <sup>-3</sup> )	12.20	2.70
$\lambda$ (Wm <sup>-1</sup> °C <sup>-1</sup> )	120	221

Table 3. Meat and cladding data

Cooling channel between tubes no.s	a (0-1)	b (1-2)	c (2-3)	d (3-4)	e (4-5)
Flow ratio $\delta_n$	0.162	0.178	0.189	0.229	0.242

Table 4. Flow distribution ratios  $\delta_n$  in a Mk 4 fuel element

Tube no.	1	2	3	4
Power deposition ratio $\Delta_n$ in a				
3-tube fuel elem.	0	0.288	0.328	0.384
4-tube fuel elem.	0.197	0.226	0.264	0.313

Table 5. Power deposition ratios  $\Delta_n$  in a Mk 4 LEU fuel element

Flowmeasurements in the DR 3 core June 12th and 13th 1986

The measurements were carried out at a D<sub>2</sub>O-temperature of 50°C, which is the normal operating temperature at 10 MW. The accuracy is estimated to be  $\pm 5\%$ . Values in the matrixes are in kg/s.

Pump combination 1P1-1 + 1P1-2; 360 kg/s

	A1: 16.2	A2: 16.0	A3: 13.8	A4: 13.2	
B1: 12.8	B2: 13.6	B3: 13.2	B4: 13.5	B5: 13.4	B6: 13.3
C1: 12.2	C2: 13.2	C3: 13.0	C4: 15.0	C5: 13.5	C6: 13.2
D1: 13.0	D2: 13.6	D3: 13.4	D4: 13.8	D5: 13.8	D6: 13.7
	E1: 15.2	E2: 15.2	E3: 13.8	E4: 13.2	Table 6a

Pump combination 1P1-2 + 1P1-3: 360.5 kg/s

	A1: 15.5	A2: 15.5	A3: 13.9	A4: 12.8	
B1: 12.5	B2: 13.7	B3: 13.2	B4: 13.5	B5: 13.7	B6: 13.6
C1: 12.0	C2: 13.3	C3: 13.2	C4: 15.1	C5: 13.5	C6: 13.3
D1: 13.0	D2: 13.7	D3: 13.7	D4: 14.0	D5: 13.8	D6: 13.2
	E1: 15.8	E2: 15.6	E3: 13.8	E4: 13.7	Table 6b

Pump combination 1P1-1 + 1P1-3: 365 kg/s

	A1: 16.2	A2: 16.0	A3: 13.9	A4: 13.5	
B1: 12.5	B2: 13.7	B3: 13.6	B4: 13.9	B5: 13.9	B6: 12.2
C1: 13.3	C2: 13.5	C3: 13.6	C4: 15.0	C5: 13.5	C6: 13.7
D1: 13.1	D2: 13.7	D3: 13.8	D4: 14.1	D5: 14.0	D6: 13.3
	E1: 15.9	E2: 15.6	E3: 13.9	E4: 13.9	Table 6c

The dotted ellipses represent the positions of the upcomers (elliptical because of the compressed delineation of the core).

Table 7a, 7b and 7c

DATA FOR LEU-FUEL ELEMENTS

Type: Mk 4 acc. to Risø drawing no. 73.33 (15. Jan. 76).  
 Enrichment: 19.75%  $^{235}\text{U}$ , balance  $^{238}\text{U}$  (+ impurities <2.0%).  
 Cladding: AlMg1 (98% Al, 1% Mg, balance Si, Fe, Cu, Mn, Cr, Zn, Ti).  
 Dimensions of plates:

Table 7a

	Thickness mm	Length mm	Width mm
Wide plates, meat	0.531	580.0	90.0
Narrow plates, meat	0.531	580.0	60.0
Cladding plates	0.464	-	-
Finished plate, wide	1.46	640.9	102.0
- - , narrow	1.46	640.9	69.0

Cover tube: Outside diameter: 103.00 mm }  
 Tube thickness: 1.50 mm } Weight of 60 cm length:  
 774.86 g  
 Thimble: Outside diameter: 54.05 mm }  
 Tube thickness: 1.63 mm } 448.38 g  
 U-density in meat: 3.29 g/cm<sup>3</sup>  
 $^{235}\text{U}$ -density in meat:  $1.66 \times 10^{21}$  at/cm<sup>3</sup>  
 $\text{U}_3\text{Si}_2$ -powder composition: <20% fine (< 45  $\mu\text{m}$ )  
 Density,  $\text{U}_3\text{Si}_2$ : 12.2 g/cm<sup>3</sup>  
 - , meat: 5.4 g/cm<sup>3</sup>  
 Heat conductivity, meat 98 W/m<sup>o</sup>C at 60<sup>o</sup>C  
 Density, cladding: 2.70 g/cm<sup>3</sup>  
 Heat conductivity, cladding: 221 W/m<sup>o</sup>C at 60<sup>o</sup>C

Table 7b

Procentage content of ↓ in →	Meat		1 cm tube in the fuel weight % <sup>zone</sup> Vol. %	
	Weight %	Vol. %	weight %	Vol. %
U	61.4		29.5	
$\text{U}_3\text{Si}_2$	66.0	29.0	31.7	9.2
Al	34.0	67.5	68.3	89.7
Porosity	-	3.5	-	1.1

Al/5

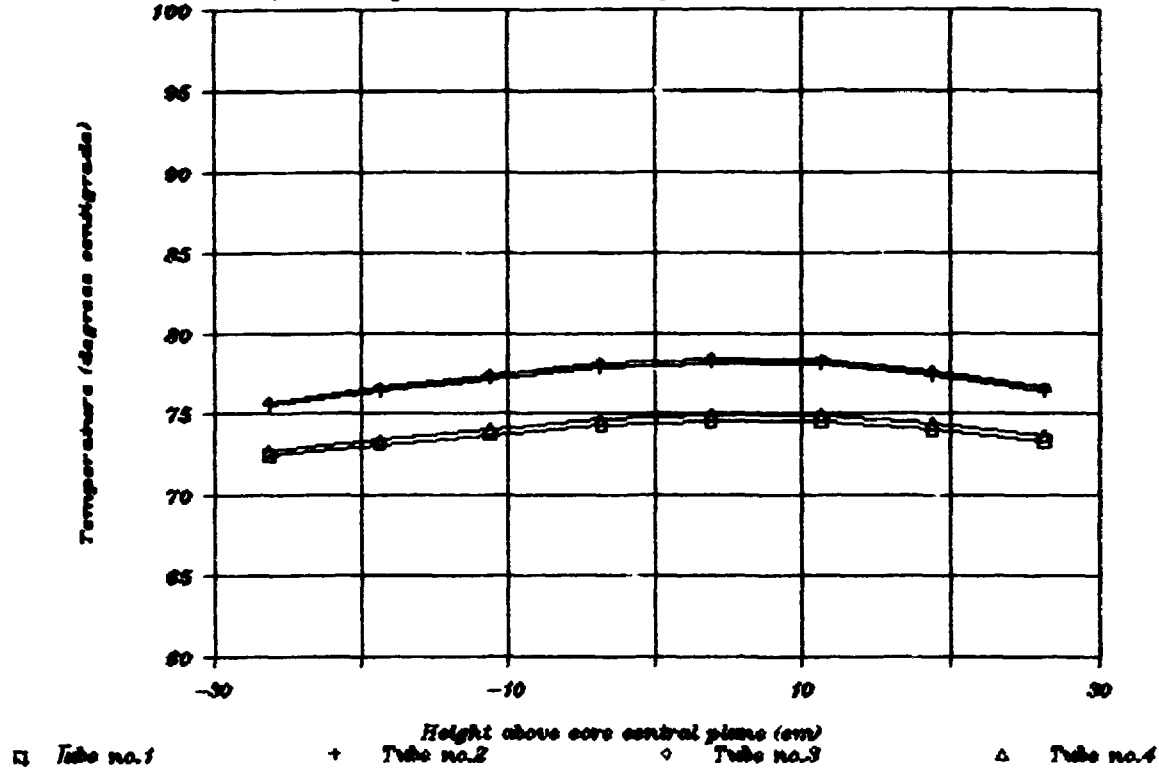
Table 7c

Tube no.	1	2	3	4	Total
<b>MEASURES (mm)</b>					
Length, tube	660.4	660.4	660.4	660.4	
- , meat	600	600	600	600	
Thickness, tube	1.50	1.50	1.50	1.50	
- , meat	0.546	0.546	0.546	0.546	
- , cladd.	0.477	0.477	0.477	0.477	
Diam., outer tube	63.85	73.65	83.45	93.25	
Angle of covering, meat	311.0°	313.5°	315.5°	317.0°	
<b>VOLUMES (cm<sup>3</sup>)</b>					
U <sub>3</sub> Si <sub>2</sub>	16.12	18.81	21.50	24.19	80.62
Al in meat	37.37	43.60	49.82	56.05	186.83
Porosity	1.94	2.26	2.59	2.91	9.70
Total, meat	55.43	64.67	73.90	83.14	277.14
Al-cladding	138.61	159.87	181.13	202.39	682.00
Total, tube	194.04	224.54	255.03	285.53	959.14
<b>WEIGHTS (g)</b>					
<sup>235</sup> U	36.0	42.0	48.0	54.0	180.0
U	182.3	212.7	243.0	273.4	911.4
Si	14.3	16.7	19.1	21.5	71.6
U <sub>3</sub> Si <sub>2</sub>	196.6	229.4	262.1	294.9	983.0
Al in meat	100.9	117.7	134.5	151.5	504.6
Total, meat	297.5	347.1	396.6	446.5	1487.7
Al-cladding	374.2	431.6	489.0	546.4	1841.2
Total, tube	671.8	778.8	885.7	992.9	3329.20
Inclusive cover tube and thimble:					4552.44
<b>SURFACE AREAS (cm<sup>2</sup>)</b>					
Tube surface	2587	2994	3400	3807	12789
- - off meat	2031	2369	2708	3046	10153
<b>COOLING CHANNEL WIDTH</b>					
(mm)					
Channel	Thimble-1	1-2	2-3	3-4	4-cover tube
Width	3.40	3.40	3.40	3.40	3.375

APPENDIX 2

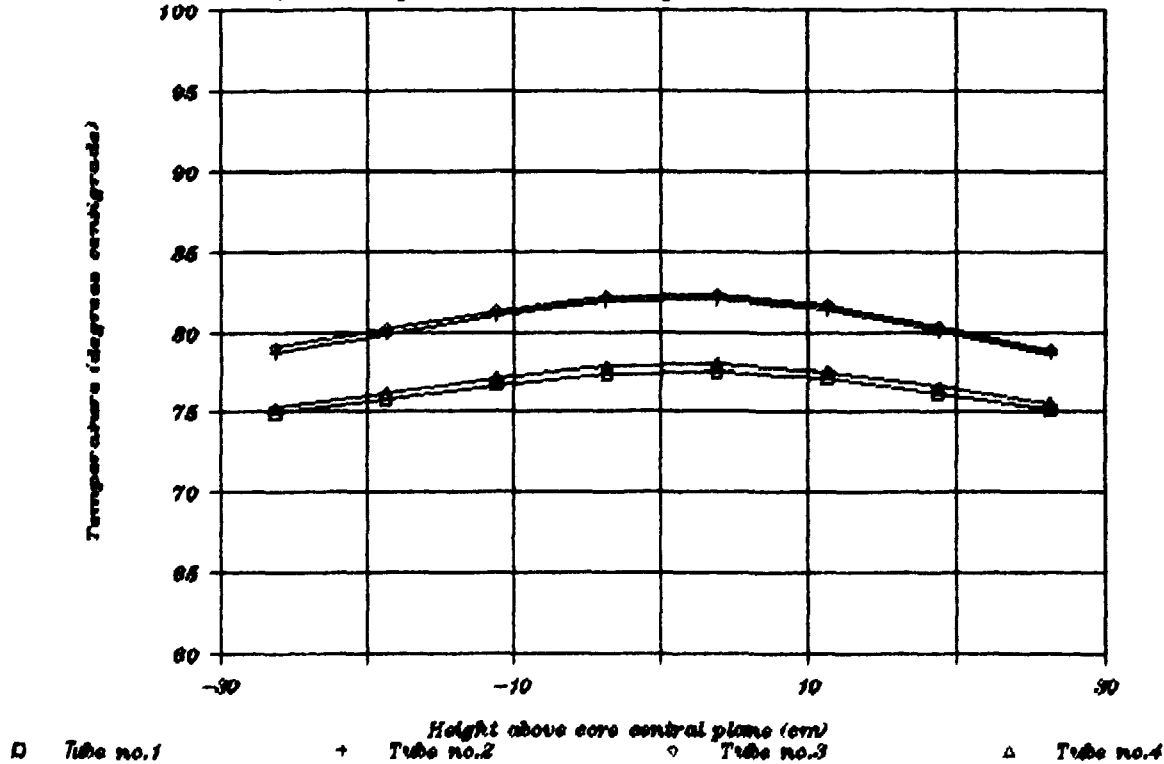
**Meat Temperatures in A1.**

DR3, Reactor cycle no.340, 303 kw, 87 ga. U235 at start D20-bulet:63.4



**Meat Temperatures in A2.**

DR3, Reactor cycle no.340, 373 kw, 87 ga. U235 at start D20-bulet:63.4



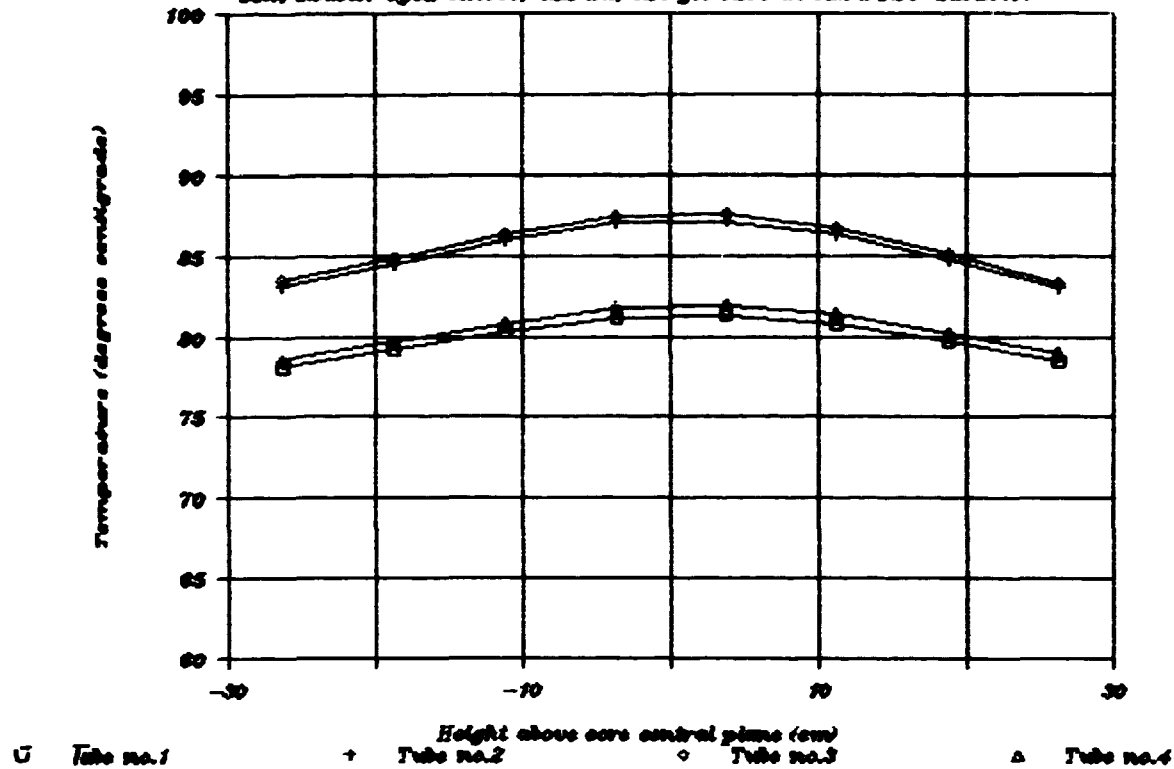
12.9.88, P.W.



A2/2

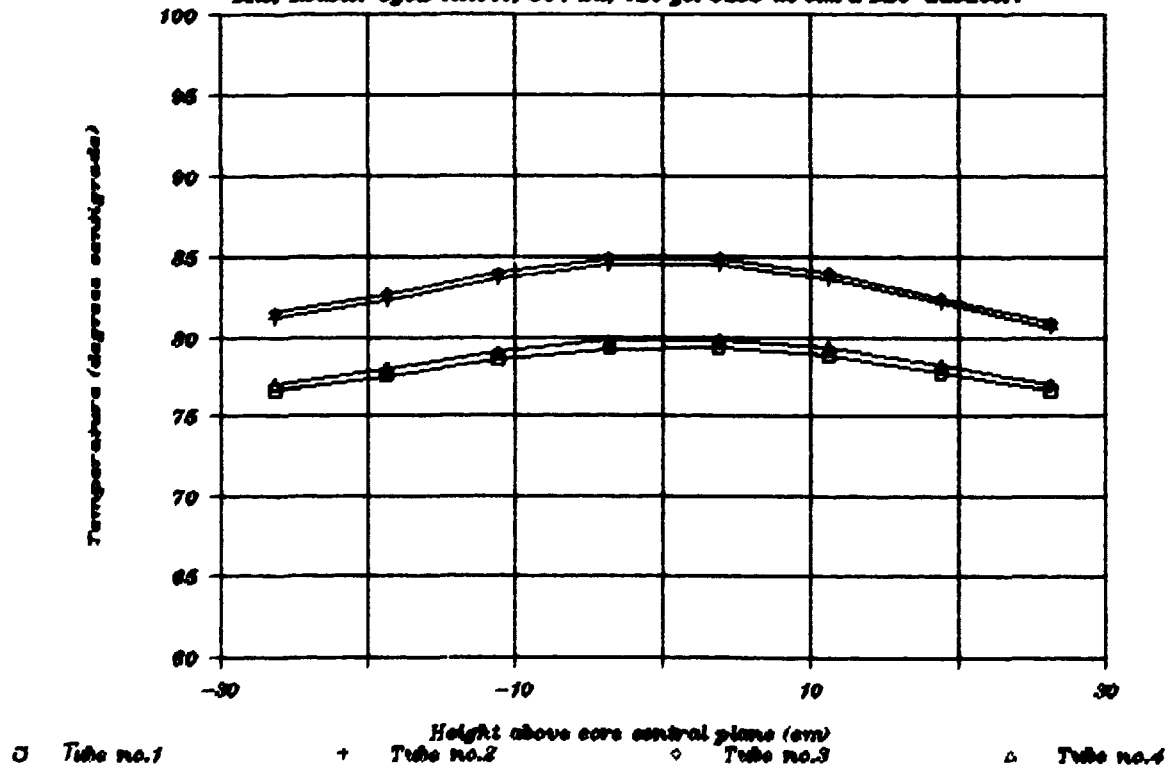
### Meat Temperatures in A3.

RR3, Reactor cycle no.340, 422 kw, 123 gm. U235 at start. B20-Inlet 63.4



### Meat Temperatures in A4.

RR3, Reactor cycle no.340, 364 kw, 126 gm. U235 at start. B20-Inlet 63.4

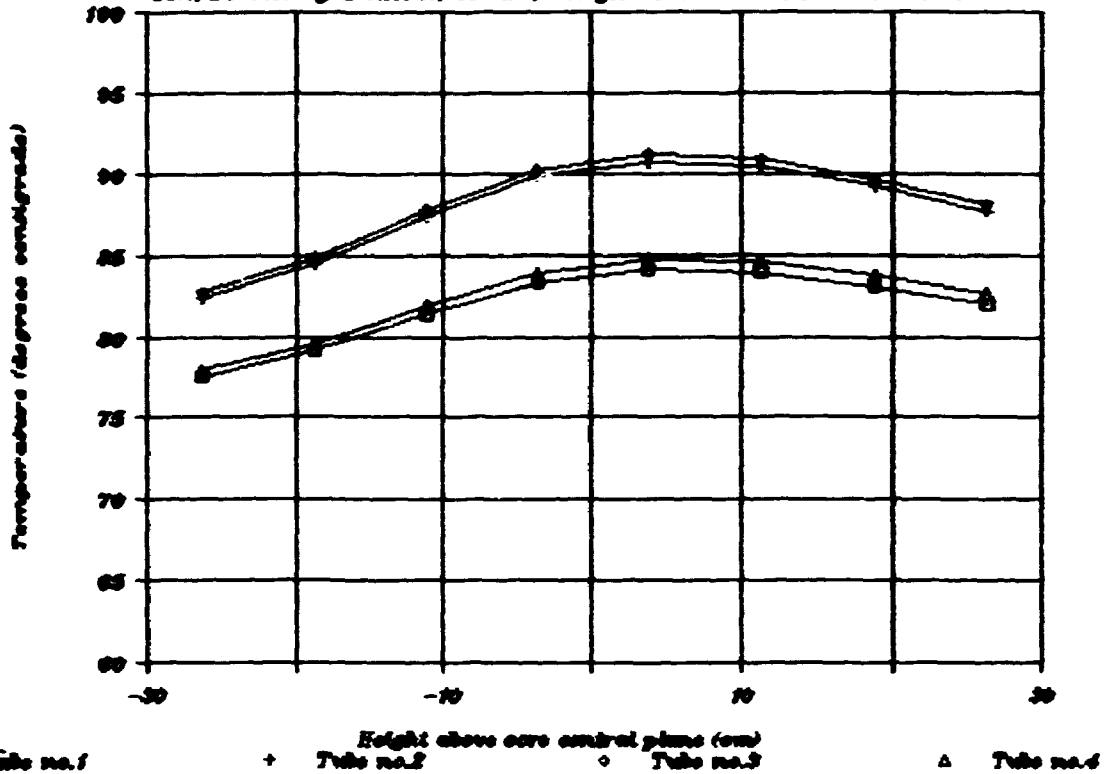


12 P. 88, 2 W.

A2/3

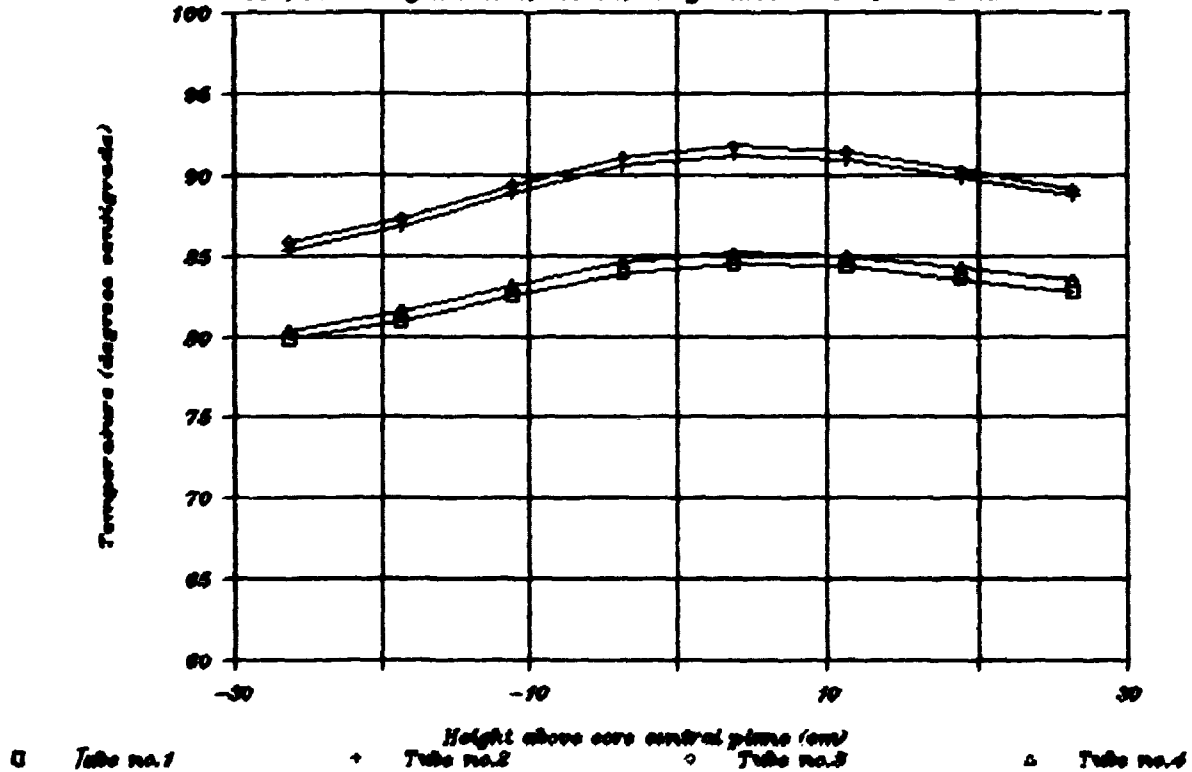
### Meat Temperatures in B1.

BR3, Reactor Cycle no.349, 437 lbs, 147 ga. U235 at start, D20-fuel: 63.4



### Meat Temperatures in B2.

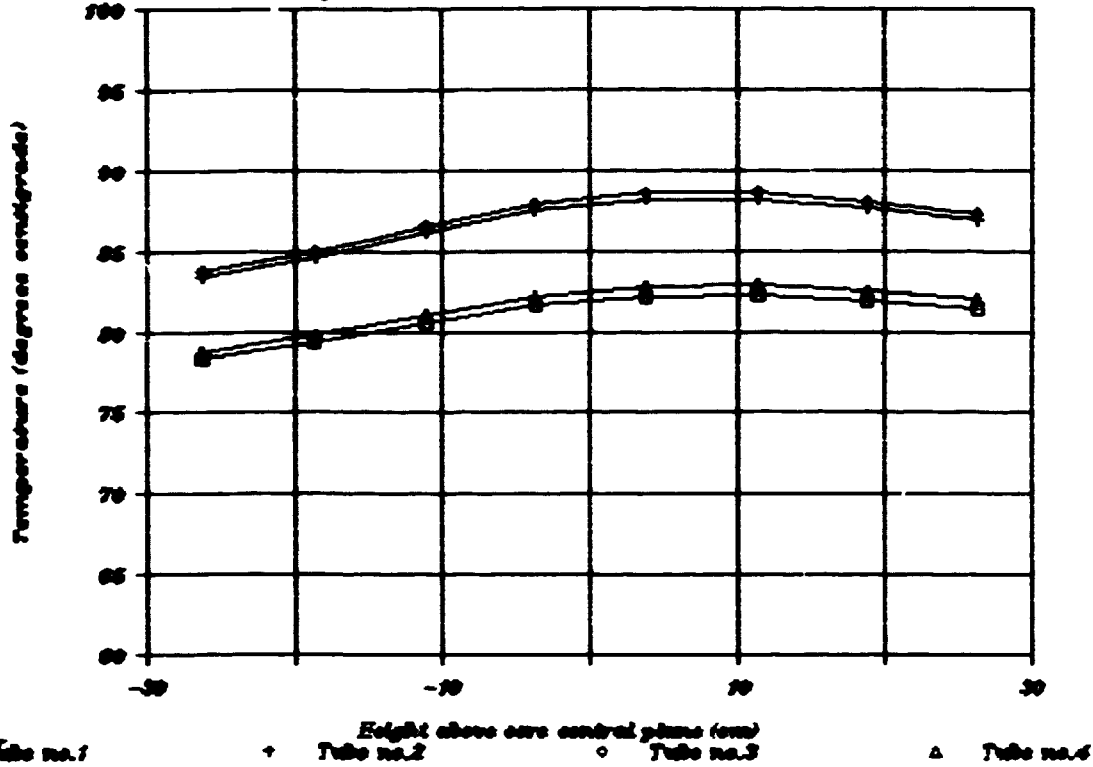
BR3, Reactor Cycle no.349, 400 lbs, 135 ga. U235 at start, D20-fuel: 63.4



12 E 8A, 7W.

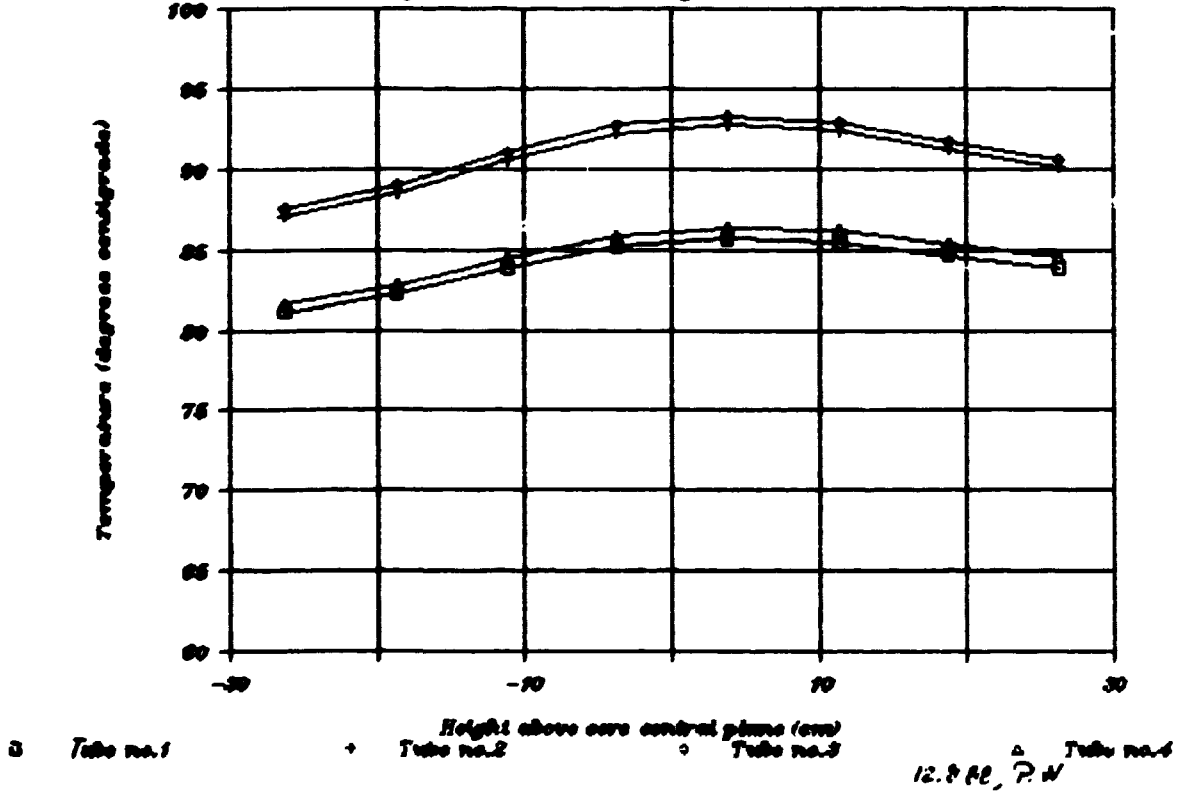
### Meat Temperatures in B3.

B23, Reactor Cycle no.340, 448 lbs. 105 gm. U235 at start. B20-641023.4



### Meat Temperatures in B4.

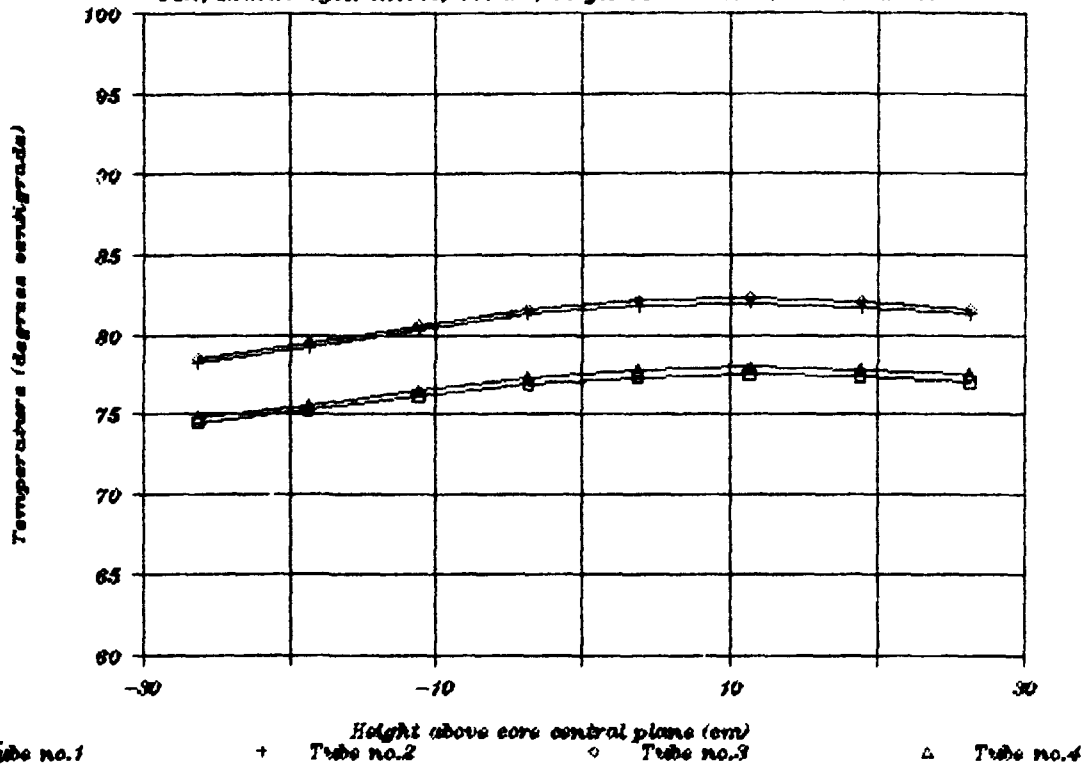
B23, Reactor Cycle no.340, 531 lbs. 135 gm. U235 at start. B20-641023.4



A2/5

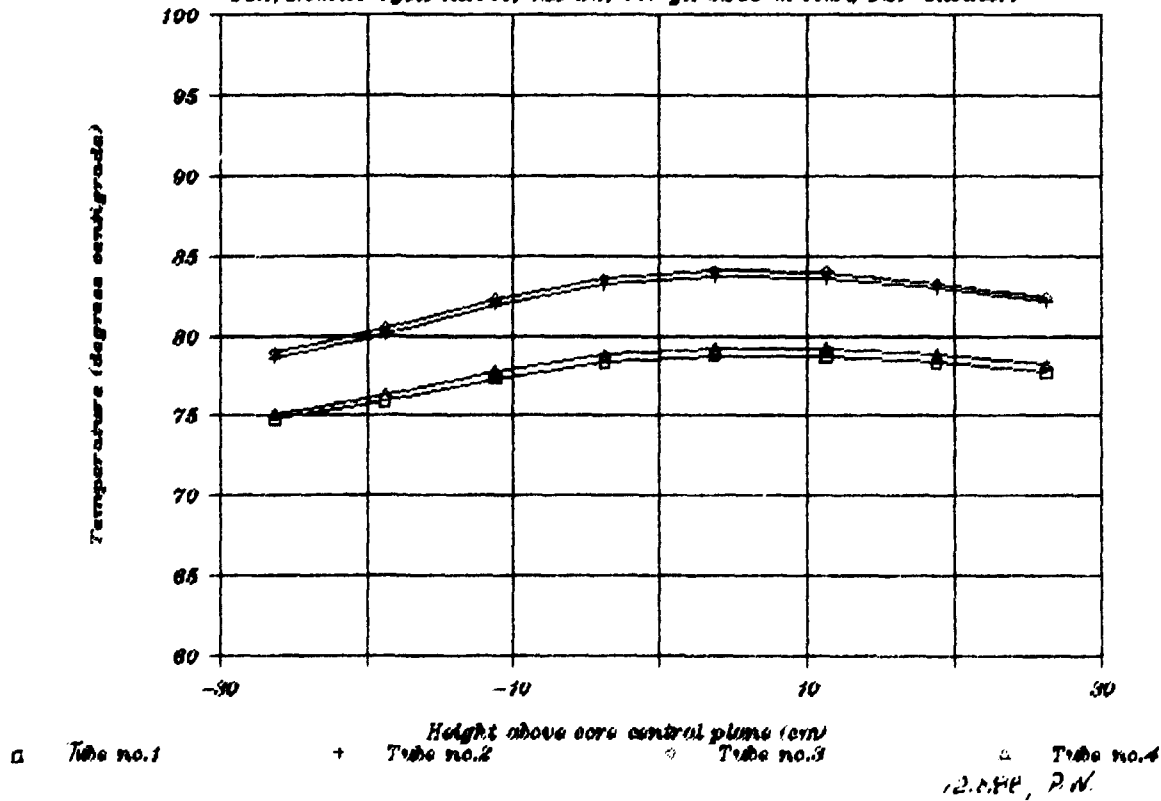
### Meat Temperatures in B5.

DR3, Reactor Cycle no. 340, 339 kw, 91 ga. U235 at start, D20-Inlet: 63.4



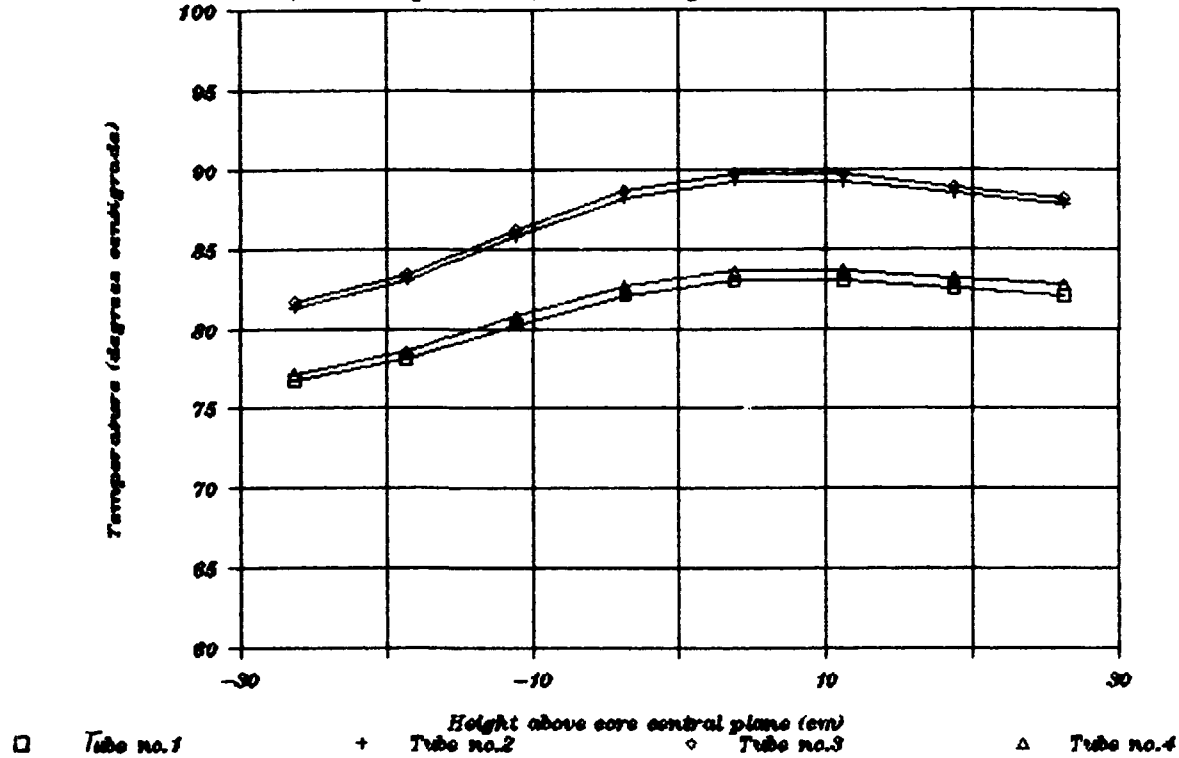
### Meat Temperatures in B6.

DR3, Reactor Cycle no. 340, 325 kw, 117 ga. U235 at start, D20-Inlet: 63.4



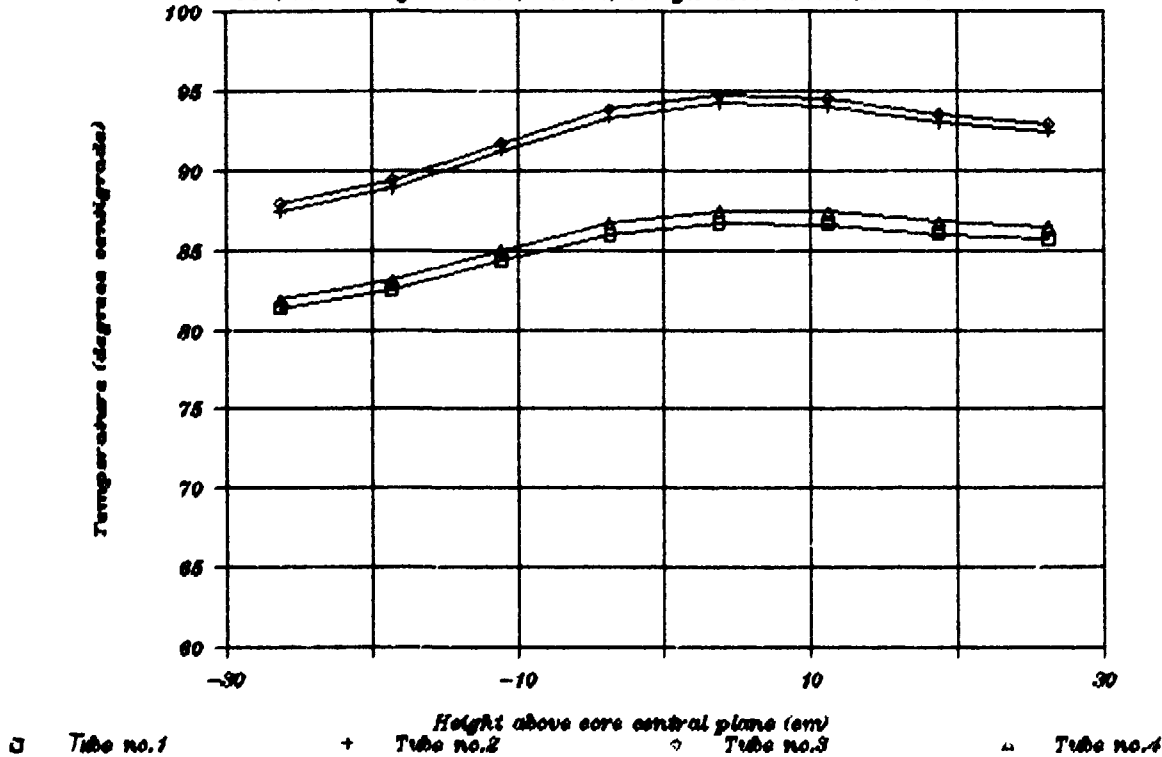
### Meat Temperatures in C1.

DR3, Reactor Cycle no.340, 440 kw, 147 ga. U235 at start, D20-inlet:83.4



### Meat Temperatures in C2.

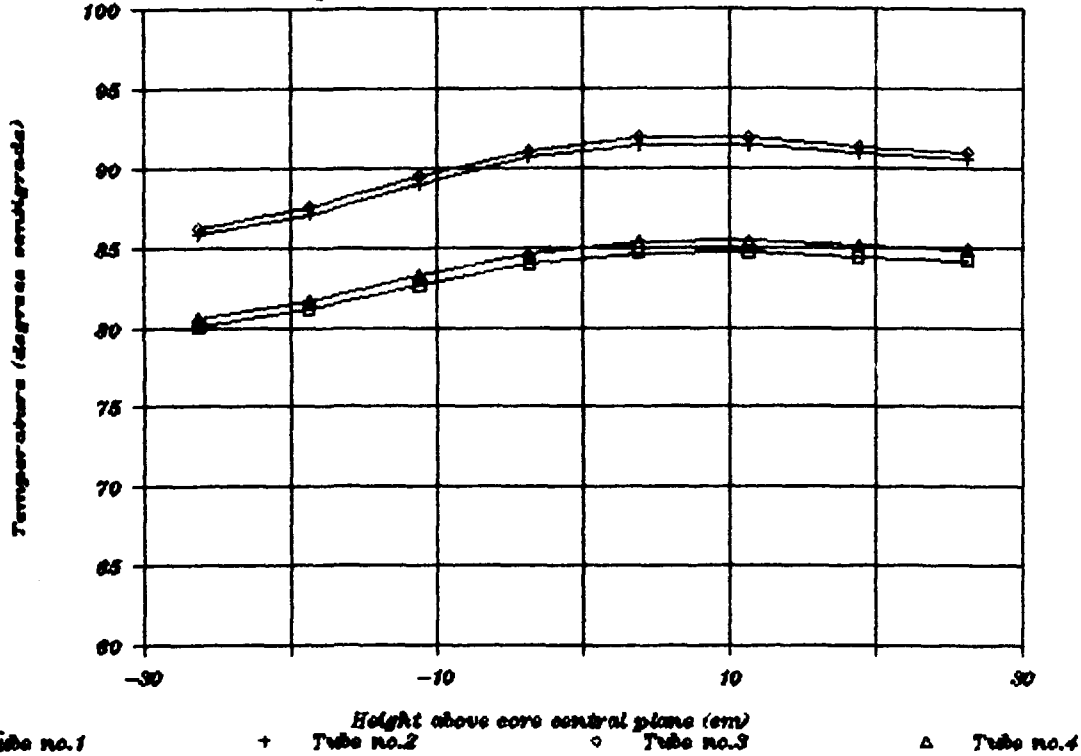
DR3, Reactor Cycle no.340, 543 kw, 147 ga. U235 at start, D20-inlet:83.4



12. A.18, 24.

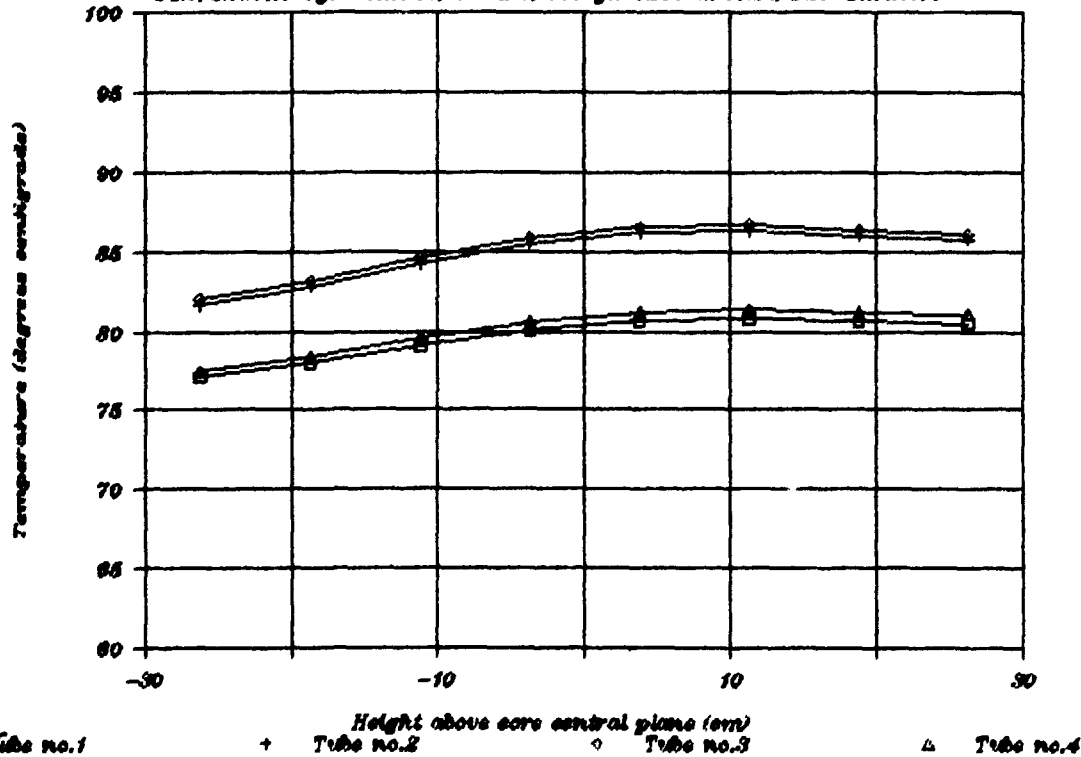
### Meat Temperatures in C3.

DR3, Reactor Cycle no.340, 507 kw, 119 ga. U235 at start, D20-inlet:63.4



### Meat Temperatures in C4.

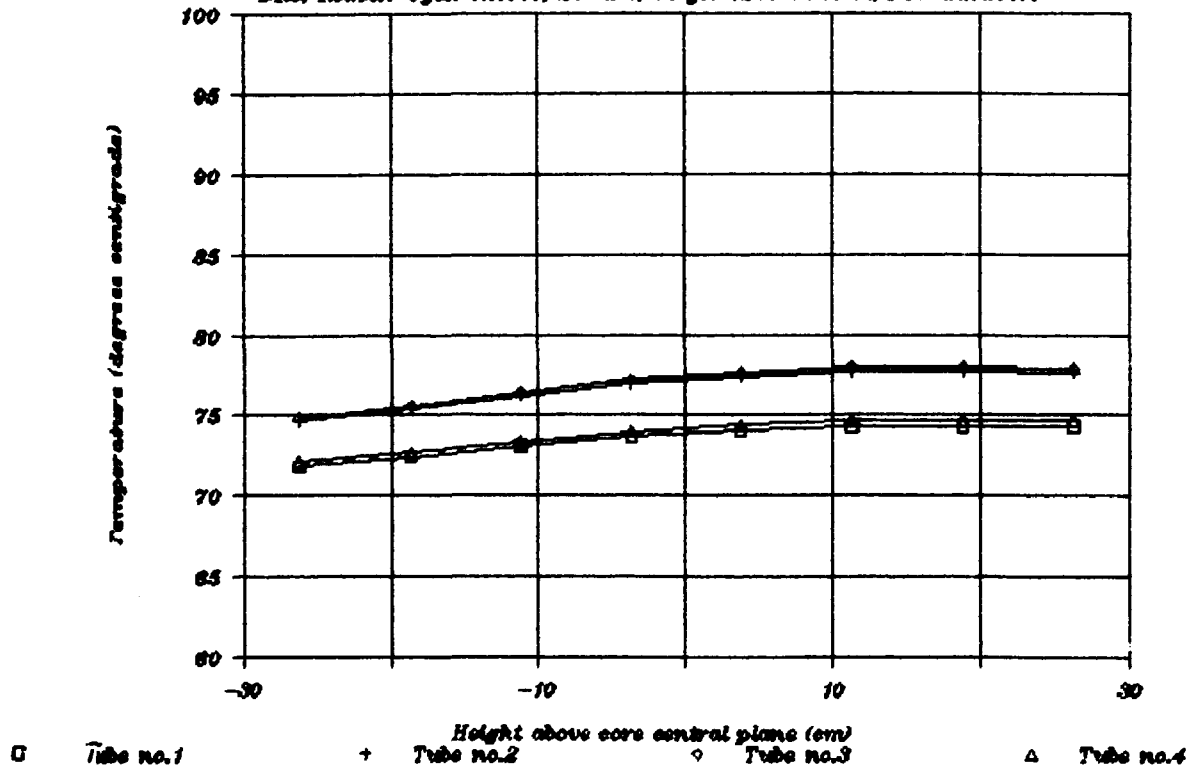
DR3, Reactor Cycle no.340, 449 kw, 109 ga. U235 at start, D20-inlet:63.4



12.8.68, P.W.

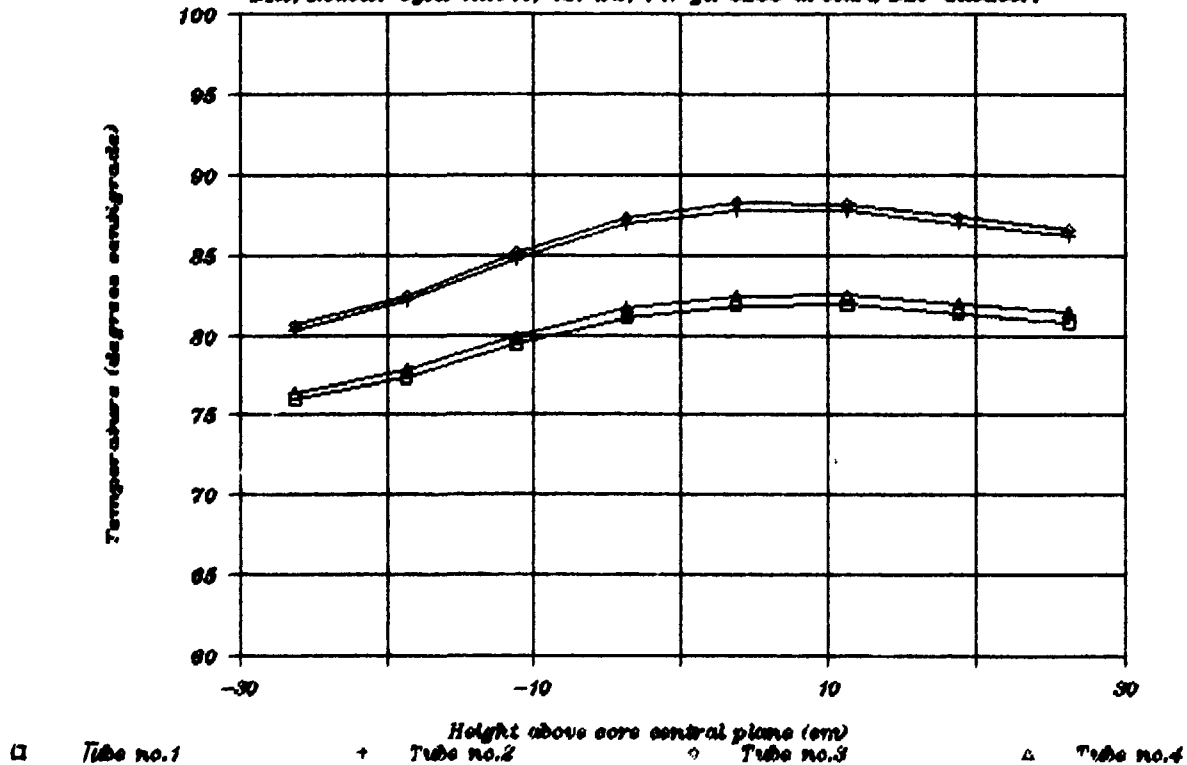
### Meat Temperatures in C5.

DR3, Reactor Cycle no.340, 287 kw, 68 ga. U235 at start, D2O-inlet:63.4



### Meat Temperatures in C6.

DR3, Reactor Cycle no.340, 421 kw, 147 ga. U235 at start, D2O-inlet:63.4

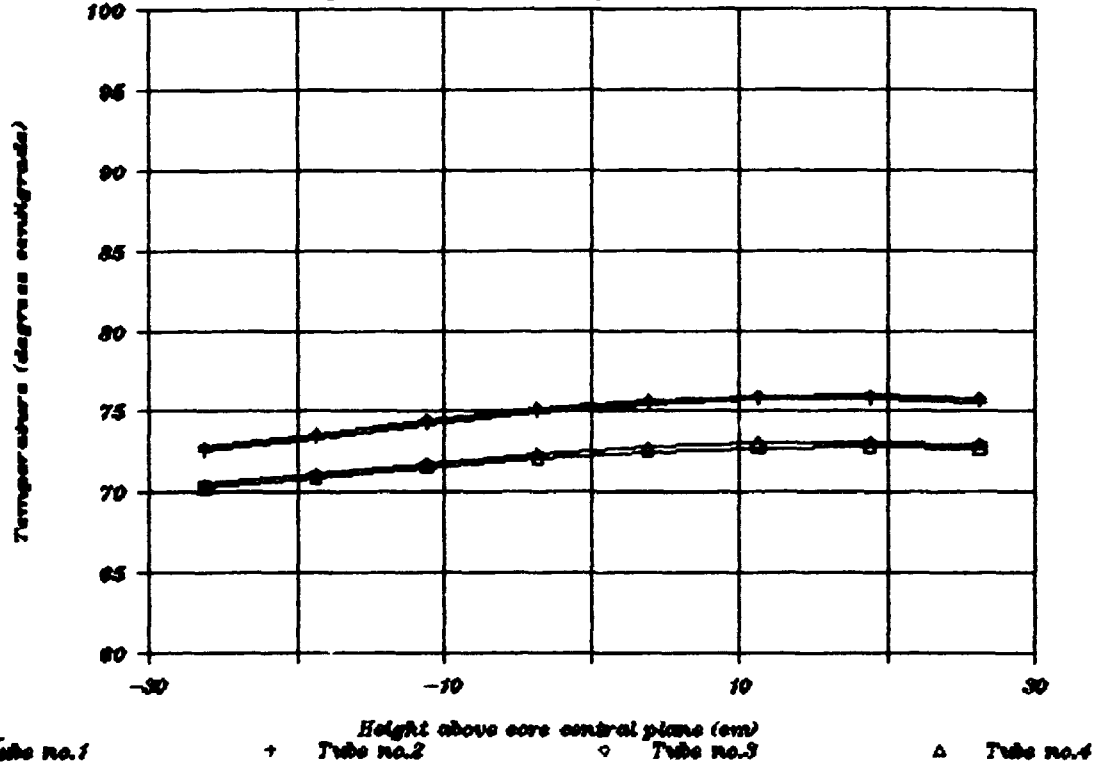


12.8.88, P.W.

A2/9

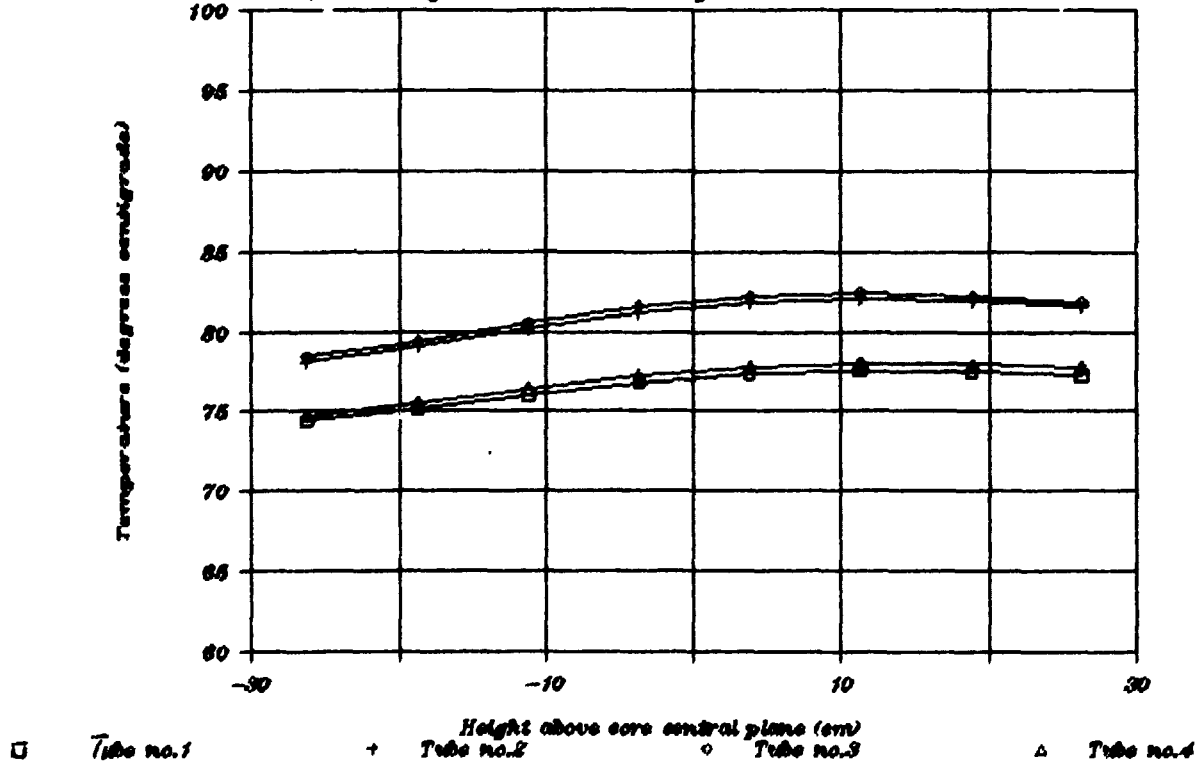
### Meat Temperatures in D1.

DR3, Reactor Cycle no. 340, 212 kw, 65 gm. U235 at start, D20-Inlet: 63.4



### Meat Temperatures in D2.

DR3, Reactor Cycle no. 340, 338 kw, 91 gm. U235 at start, D20-Inlet: 63.4



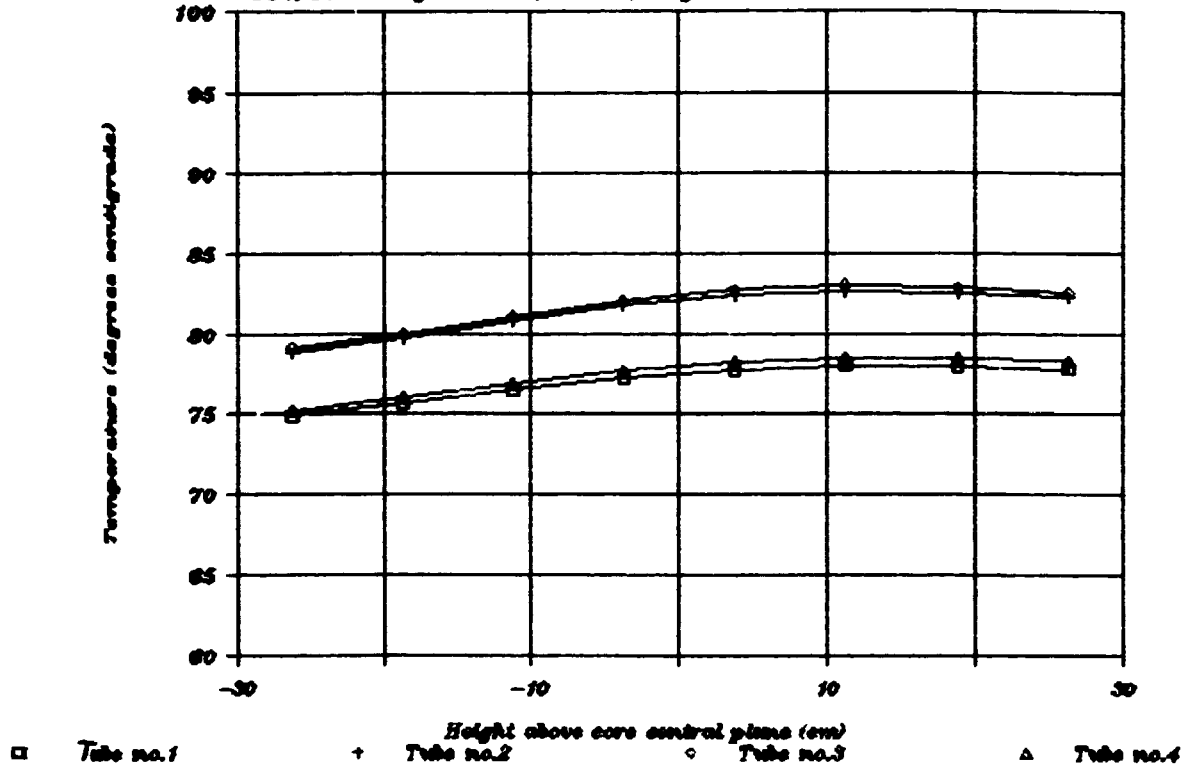
12.8.88, P.W.



A2/10

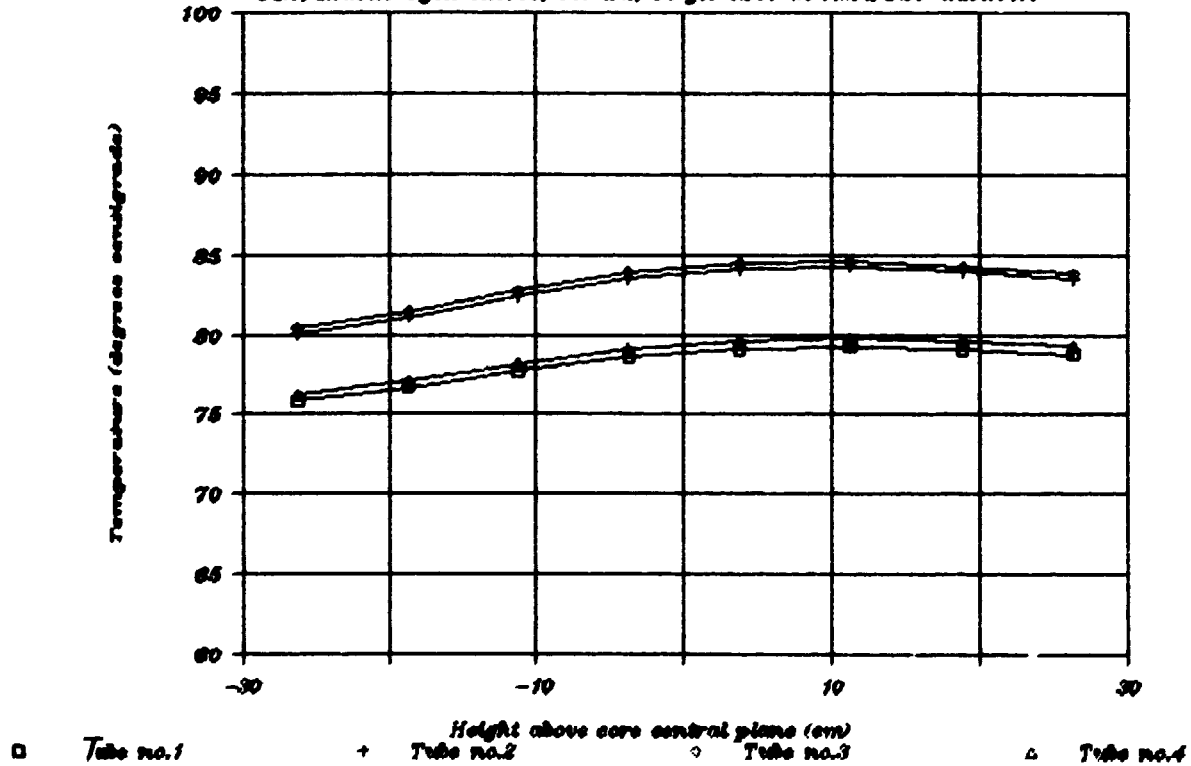
### Meat Temperatures in D3.

BR3, Reactor Cycle no.340, 351 hrs, 88 ga. U235 at start, D20-inlet:63.4



### Meat Temperatures in D4.

BR3, Reactor Cycle no.340, 357 hrs, 88 ga. U235 at start, D20-inlet:63.4

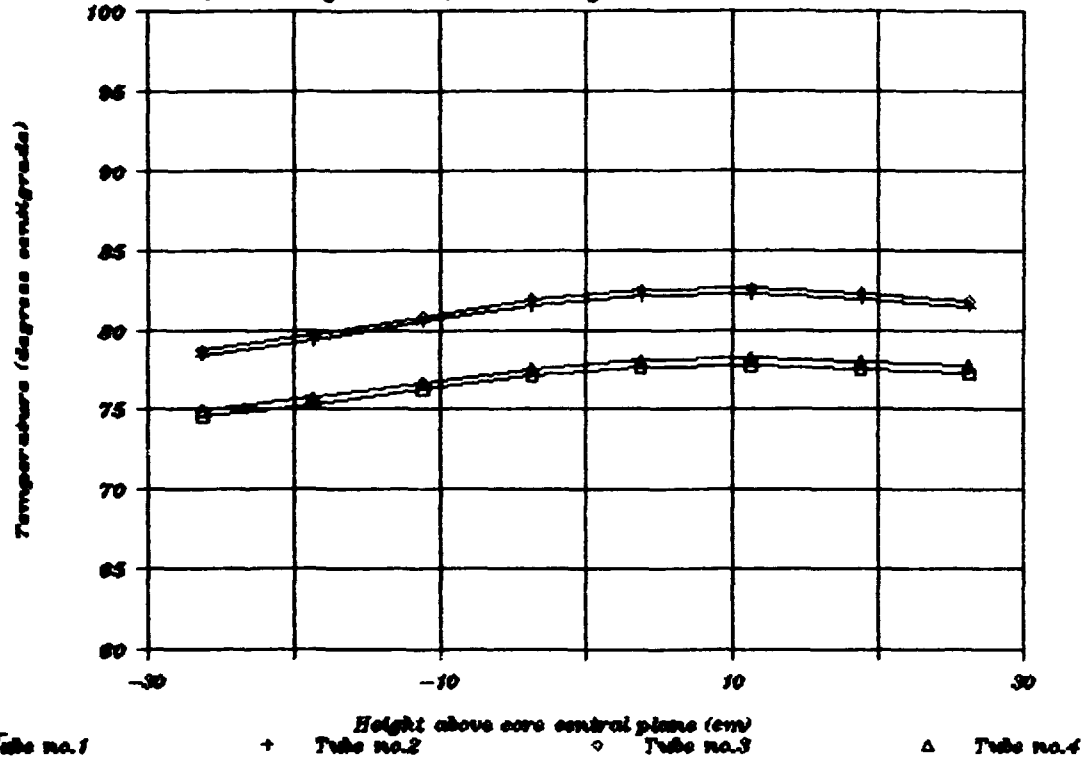


12.6 KF, P. W.

A2/11

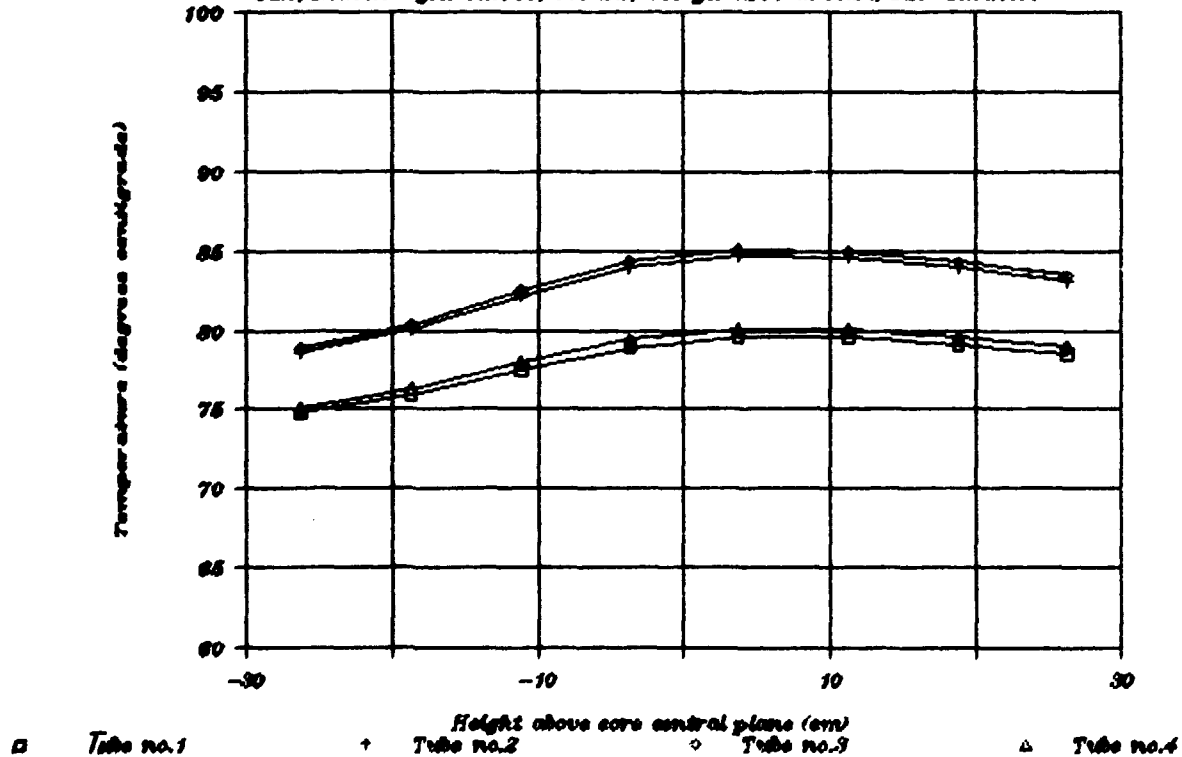
### Meat Temperatures in D5.

DR3, Reactor Cycle no.340, 344 kw, 99 gm. U235 at start, D20-inlet:63.4



### Meat Temperatures in D6.

DR3, Reactor Cycle no.340, 360 kw, 138 gm. U235 at start, D20-inlet:63.4

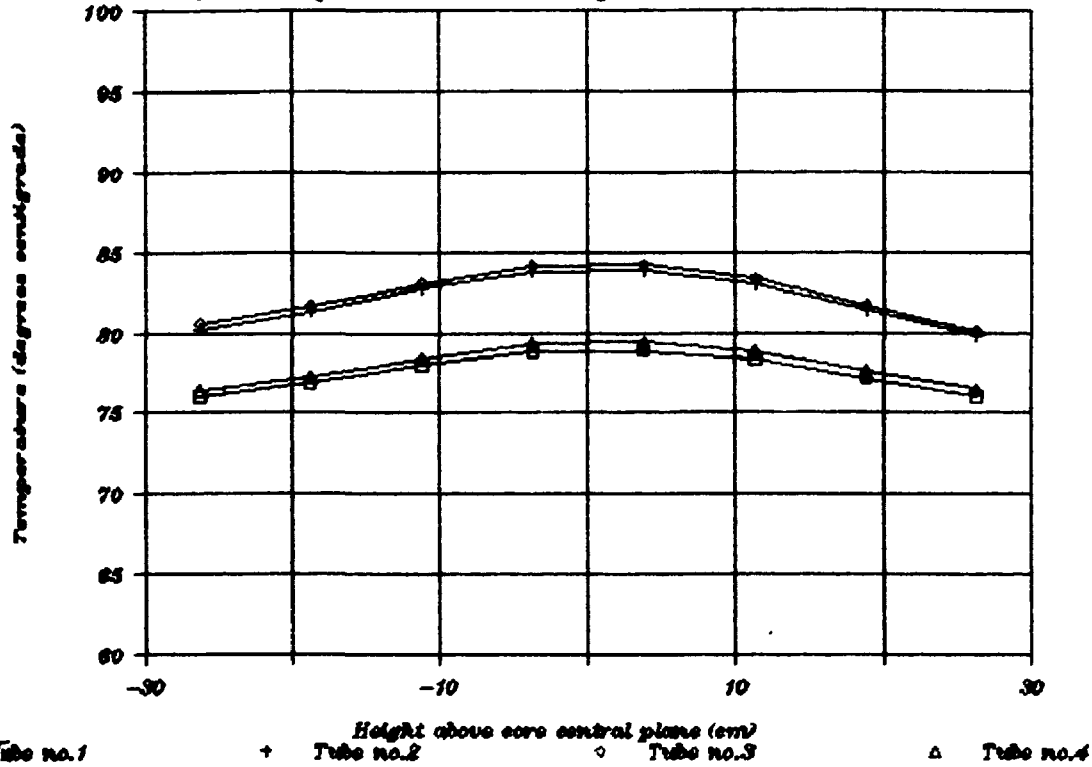


12.8.66, P. W.

A2/12

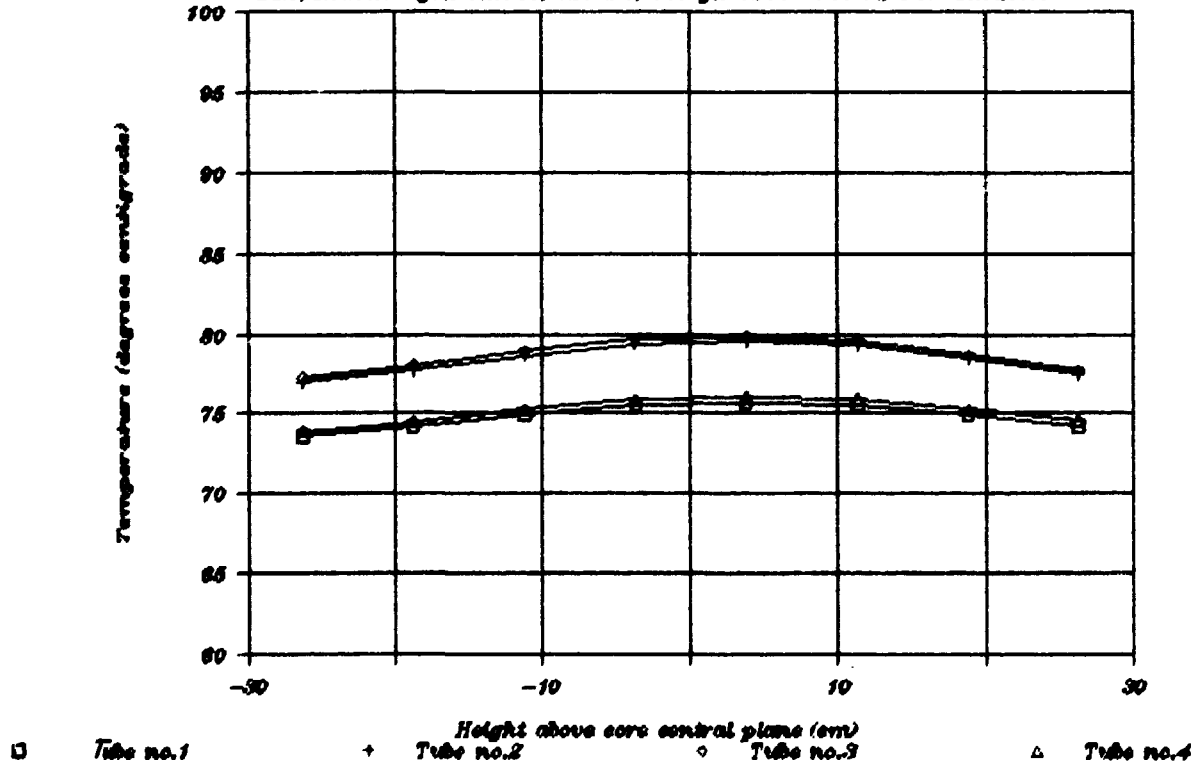
### Meat Temperatures in E1.

DE3, Reactor Cycle no.340, 400 kw, 147 ga. U235 at start, D20-inlet:63.4



### Meat Temperatures in E2.

DE3, Reactor Cycle no.340, 320 kw, 103 ga. U235 at start, D20-inlet:63.4

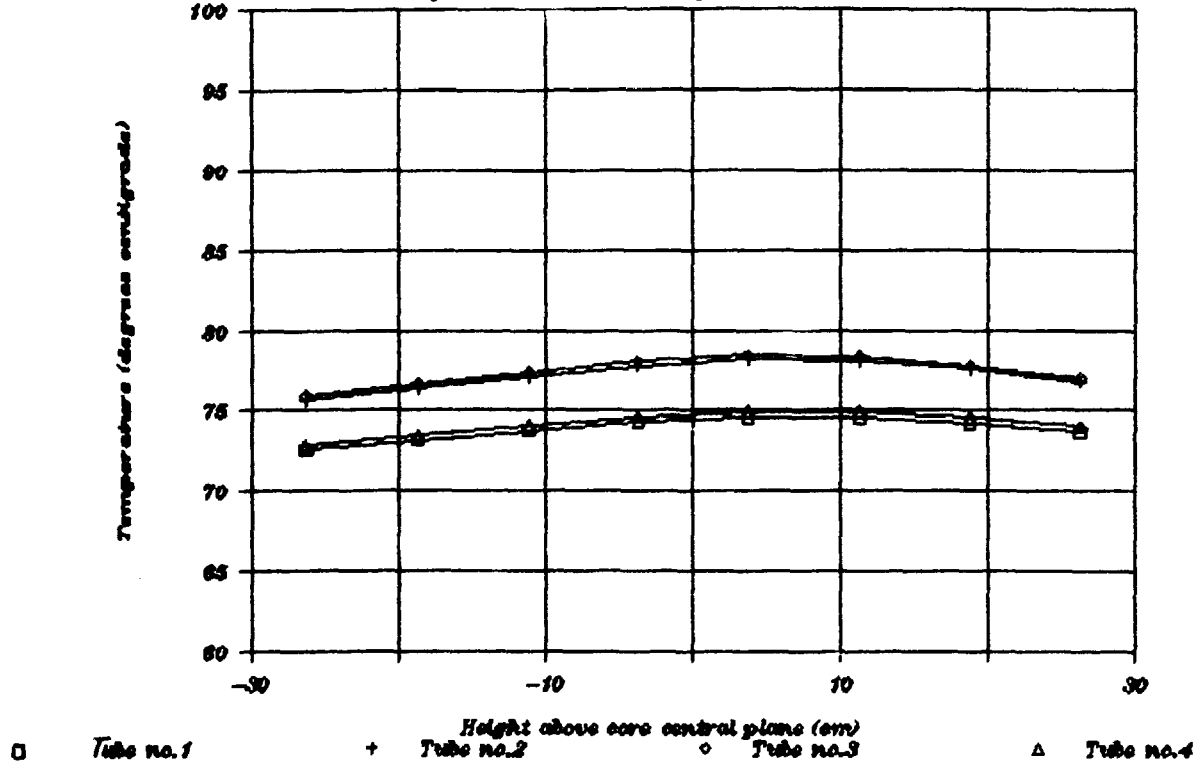


12.8.88, P.V.

A2/13

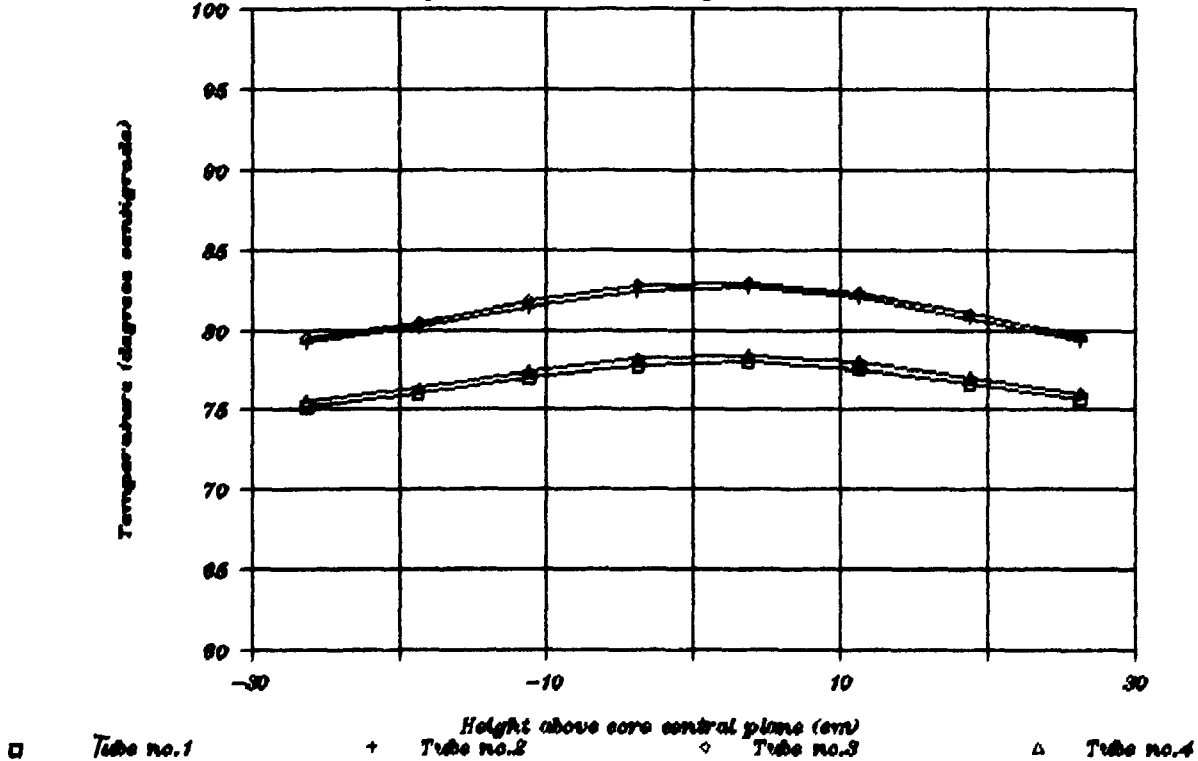
### Meat Temperatures in E3.

DR3, Reactor Cycle no. 340, 288 kw, 85 ga. U235 at start, D20-Inlet: 63.4



### Meat Temperatures in E4.

DR3, Reactor Cycle no. 340, 338 kw, 128 ga. U235 at start, D20-Inlet: 63.4



12.8.88, P.W.

<b>Title and author(s)</b>  Calculation of plate temperatures in a Mk 4 LEU fuel element  by  Karsten Haack	<b>Date</b> September 1988
	<b>Department or group</b>  Reactor DR 3
	<b>Groups own registration number(s)</b>  18/N 219
	<b>Project/contract no.</b>  
<b>Pages</b> 38 <b>Tables</b> 11 <b>Illustrations</b> 32 <b>References</b> 8	<b>ISBN</b> 87-550-1462-3
<b>Abstract (Max. 2000 char.)</b>  A calculation method for estimating the axial temperature distributions of each tube in each of the 26 fuel elements of the DR 3 core is described and demonstrated. With input data for fuel element power, D <sub>2</sub> O outlet temperature and main D <sub>2</sub> O circulator combination, a computer code calculates all important temperatures in the fuel element.	
<b>Descriptors - INIS</b> COMPUTER CALCULATIONS; DR-3 REACTOR; FUEL ELEMENTS; MEDIUM TEMPERATURE; MODERATELY ENRICHED URANIUM; SPECIFICATIONS; TEMPERATURE DISTRIBUTION	
Available on request from Rise Library, Rise National Laboratory, (Rise Bibliotek, Forskningscenter Rise), P.O. Box 40, DK-4000 Roskilde, Denmark. Telephone 02 37 12 12, ext. 2202. Telex: 43116, Telefax: 02 36 06 00	

**Available on request from  
Risø Library,  
Risø National Laboratory, P.O. Box 49,  
DK-4000 Roskilde, Denmark  
Phone (02) 37 12 12 ext. 2262**

**ISBN 87-550-1462-3  
ISSN 0418-6435**